

# **URANIUM-THORIUM LIQUID-SALT REACTOR FOR PRODUCING VALUABLE RADIOISOTOPES**

**Alexander DeVolpi (retired, Argonne National Laboratory)  
and  
Ralph Moir (retired, Lawrence Livermore National Laboratory)**

**TEAC5, Chicago, IL, May 2013**

## **BOTTOM LINE**

- Valuable radioisotopes – such as tritium, He-3, Mo-99, and Np-237
- could be produced and separated most efficiently and profitably
  - >> in a single,
  - >> small,
  - >> fluid thorium or uranium fuel,
  - >> liquid-salt-cooled reactor (LSR)
  - >> most likely situated on a government reservation

## **SCOPE OF PRESENTATION**

- **Survey of current and changing status**
- **For such valuable byproducts, specifically:**
  - >> **utilization,**
  - >> **supply,**
  - >> **shortages,**
  - >> **and production options**
- **Ralph Moir and I gave a longer, more technical version of this presentation**
  - >> **yesterday**
  - >> **at Argonne National Laboratory**

# **RADIOISOTOPE PRODUCTION AND APPLICATIONS**

- **Some radioisotopes indispensable**
  - >> **for applications in medicine, industrial research, nuclear weapons, national security, and outer space**
- **Special considerations**
  - >> **their rarity,**
  - >> **production cost,**
  - >> **and handling requirements**
- **Radioisotopes are valuable commodities on the open market**

## **MOLYBDENUM-99 RADIOISOTOPE**

- **The most common medical radioisotope**

- >> **technetium-99**

- >> **half-life 6 hours** (short, medically-efficient lifetime)

- **~ 30 million medical procedures per year**

- >> **accounting for 80% of all nuclear diagnostic procedures worldwide.**

- **Derived from**

- >> **molybdenum-99 nuclear-reactor fission product**

- >> **half-life 66 hours** (manageable lifetime for production and shipment)

# **PRODUCTION OF RADIOISOTOPES**

- **Almost all diagnostic and therapeutic radioisotopes**
- **Produced in nuclear reactors,**
  - >> **mostly being byproducts of the fission process**
  - >> **either in reactor fuel**
  - >> **or in specifically designed targets**
  - >> **that contain fissile materials**
- **Nuclear accelerators have a small specialized role**

## **NUCLEAR-WEAPON TRITIUM REQUIREMENTS**

- **Require tritium**

- >> **that must be replenished**
- >> **after an unknown fraction — perhaps half —**
- >> **of tritium having decayed with its 12.3 year half-life**

- **In addition,**

- >> **research and development**
- >> **of controlled fusion calls for considerable tritium**

## **HELIUM-3**

- **The decay product of tritium is the very rare gas helium-3**
  - >> **Supply dependent on recovery from nuclear weapons**
  - >> **(Not the same issue as shortage in conventional helium gas)**
- **Helium-3 especially useful now in neutron detectors,**
  - >> **especially those deployed for homeland security throughout the world**
- **Also important basic research applications**



## **Plutonium-238**

- **The heavy non-fissile radioisotope plutonium-238**
  - >> **produced from neptunium-237 in high-power reactors,**
  - >> **ideal for thermoelectric generators,**
    - especially for long missions in outer space**
  - >> **not usable for nuclear explosives**
  
- **In U.S. national-security and non-proliferation restrictions**
  - >> **for radioisotopes tritium, helium-3, and Plutonium-238**
  - >> **require government control and processing**
  - >> **in government facilities**

# **NATIONAL-SECURITY AND NON-PROLIFERATION CONSTRAINTS**

- **Radioisotopes tritium, helium-3, and plutonium-238**

- >> **require government control and processing**

- >> **in government facilities**

## **SHORTFALLS IN MEDICAL-RADIOISOTOPE PRODUCTION**

- **Ongoing and emerging shortfalls**
  - >> **in supplies of valuable radioisotopes**
  - >> **recognized by various international and national commissions**
- **At present, there are no major producers in the United States**
  - >> **of molybdenum-99 for medical use**
  - >> **for medical use in the United States**
- **Most of current molybdenum-99 world production**
  - >> **by inefficient irradiation of solid targets**
  - >> **low-enriched uranium**
  - >> **in research and test reactors**

## **LONGER-TERM RADIOISOTOPE SHORTFALLS**

- **In the distant future:**

- >> **additional demand likely from thermonuclear fusion breeders**
- >> **require a large tritium inventory for startup.**

- **Near term:**

- > **United States and the surrounding world**
- > **far more viable applications than forthcoming supply**
- > **for these and some other rare radioisotopes.**

## **PROPOSED LIQUID-SALT REACTOR FOR RADIOISOTOPE PRODUCTION**

- **Small nuclear reactor proposed**
  - >> **to supply specialized radioisotopes**
  - >> **in a timely, cost-effective, and secure manner**
- **Mixed liquid-salt combination of fuel and coolant.**
  - >> **reactor could be quite similar in many respects**
  - >> **to circulating molten-salt reactors**
  - >> **developed at Oak Ridge National Laboratory**
- **Mutually constructive role**
  - >> **liquid-salt reactor**
  - >> **small, modular**
  - >> **fissile/fertile fuel adaptable**
  - >> **conservative design parameters**
  - >> **abundant radioisotope production**

## **GOVERNMENT SITING OF FACILITY**

- **To meet near-term requirements**
- **New, dedicated facility**
  - >> **in a remote area**
  - >> **on a government reservation**
  - >> **with state-of-the-art safety and security features**
- **Government siting recommended**
  - >> **for timeliness and national security**
  - >> **more rapid licensing of reactor**
  - >> **lower cost because of reduced construction delay**
  - >> **non-government organizations supportive**

## **ABOUT MOLTEN-SALT REACTORS**

- **Basically simple, reliable nuclear reactor**
  - >> **can function at low, near-atmospheric pressures**
  - >> **reduced mechanical stress endured by the system**
  - >> **simplified reactor design, improved safety**
- **Oak Ridge researched liquid-fueled and cooled reactors up through the 1960s**
  - >> **included uranium-233 and thorium**

# **MOLTEN-SALT BREEDER REACTOR EXPERIMENT**

- **Oak Ridge work culminated with the Molten-Salt Reactor Experiment**

- >> **7.4 Megawatt (thermal) test reactor**
- >> **started operation in 1965**
- >> **operated safely and reliably**
- >> **maintained without excessive difficulty.**
- >> **one-fluid reactor**
- >> **four-year experiment, about 1.5 years of full-power operation.**

- **Simulated basic neutronic characteristics**

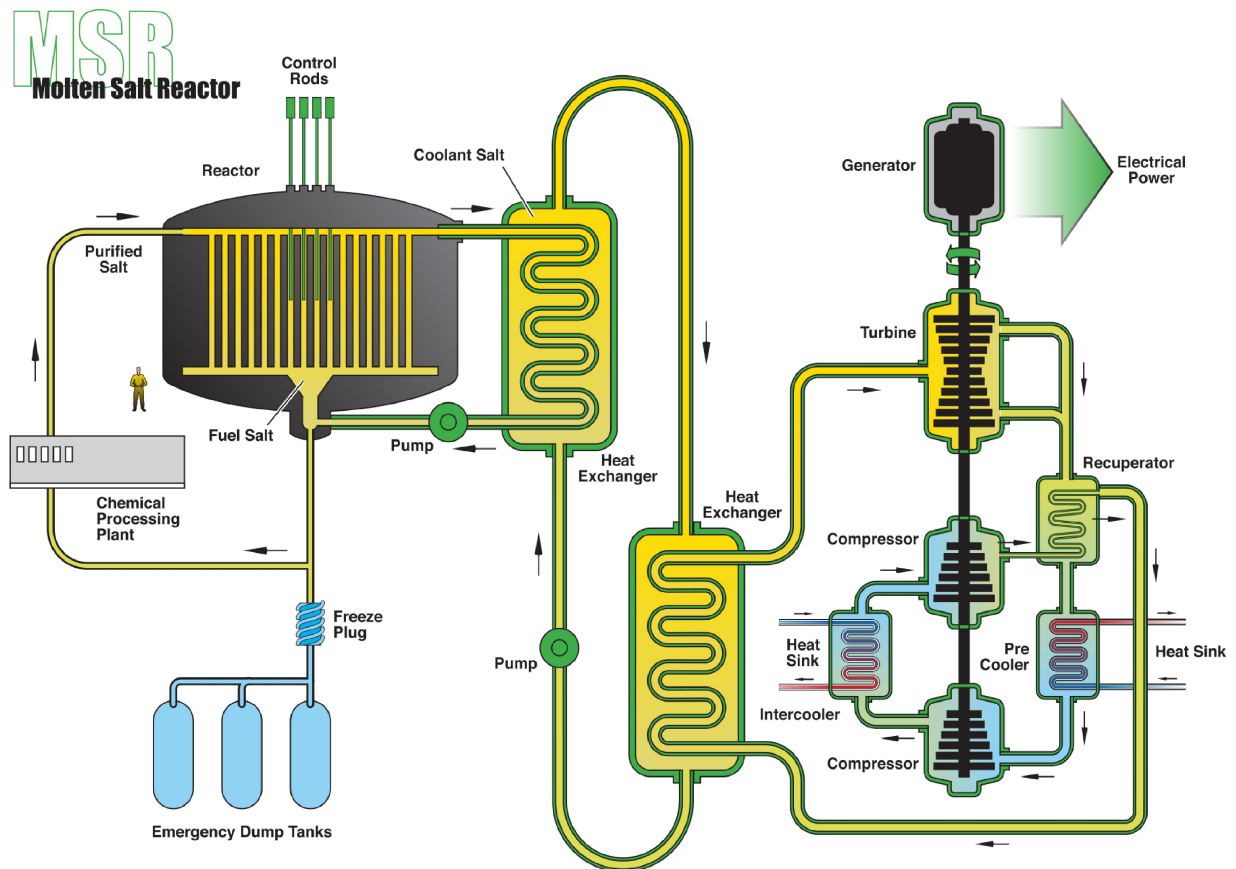
- >> **epithermal liquid-fluoride thorium breeder reactor.**

- **Primarily two fuels:**

- >> **first uranium-235**
- >> **later uranium-233 bred from thorium in other reactors**



# SCHEMATIC OF MSRE HEAT-CIRCULATION AND CHEMICAL-PROCESSING



**Figure 1.** Schematic diagram of a molten-salt reactor based on 1960's experiment at Oak Ridge.

## **KEY FEATURES OF MSRE**

- **Graphite moderator**
- **Continuous circulation of molten salt**
- **Online, continuous chemical processing**
  - >> **gaseous and solid fission products**
  - >> **other extraneous coolant-transported materials**
- **Solid control/safety rods**
- **Additional safety with a drain plug**
  - >> **kept solid by actively freezing plug**
  - >> **provides passive safety system**
  - >> **in case of electric-supply failure or overheating**
  - >> **would melt and drain solution into “nuclear-safe” geometries**

## **FLIBE**

- **A fluid solution used as a coolant and carrier**
  - >> **FLIBE = FLuorine - LITHium - BEryllium**
  - >> **chemically compatible mixture**
  - >> **of liquid salts**
- **Oak Ridge MSRE reactor combined primary coolant and fuel**
  - >> **FLIBE and uranium**
- **MSRE secondary coolant was FLIBE**
  - >> **provides very good, reactor-compatible properties**
- **FLIBE lithium content needed for tritium production**
- **FLIBE acts as solvent and carrier for tritium and fission products**
  - >> **for on-line, continuous chemical extraction**
  - >> **optimum efficiency for high yield production**

## **AIRCRAFT REACTOR EXPERIMENT**

- **First molten-salt reactor**
  - >> **at Oak Ridge in early, mid-1950s**
  - >> **2.5 MW(th)**
  - >> **military experiment designed to attain a high power density**
  - >> **for use as an engine in a nuclear-powered bomber**
- **Used molten-fluoride salt in core zone**
  - >> **and liquid sodium as a secondary coolant**
- **Operated successfully and sufficiently for a 1000-hour cycle in 1954**

## **FEATURES OF NEW PROPOSAL FOR PRODUCTION OF VALUABLE RADIOISOTOPES**

- **A single solution-type reactor**
  - >> **specializing in radioactive-materials production**
  - >> **short-lifetime fission products, extracted in as brief a time as possible**
- **Tritium production in FLIBE**
- **Yield of other commercial radioisotopes**
  - >> **should effectively ease the forthcoming shortfall**
  - >> **at reduced cost of valuable medical radioisotopes**
- **Np-237 can be extracted too**
  - >> **to make valuable Plutonium-238 used for thermoelectric generators**

## **ESTIMATING COMMERCIAL VALUE**

- **rough estimates of the annual commercial value and government savings**
  - >> **70% capacity factor**
  - >> **100MW(thermal)**
- **From tritium production of about 210g/yr**
  - >> **valued at \$40,000/g**
  - >> **offsets significant portion of current federal tritium budget outlay**
  - >> **cost significantly less than current government production**
- **Molybdenum-99: 50 g/yr**
  - >> **At 0.5MCi/g, this would correspond to 340 MCi/yr**
  - >> **Assuming a 1% extraction yield in processing**
  - >> **at a price of \$200/Ci (for a 6-day Curie),**
- **Tritium extraction (and He-3 accumulation) simultaneous with Mo-99 production**
  - >> **respective product yields fully independent of each other**
  - >> **government budget outlays significant**
- **Additional market value for heat and power produced**

## **SUMMARY OF PRODUCTION EXPECTATIONS**

- **Tritium production for U.S. government**

**~210g/yr → ~\$8.4M/yr**

- **molybdenum-99 6-day-Curie fission-product market value**

**~50g/yr → ~\$140M/yr**

- **Marketable power**

**~100MWth → ~\$12M/yr**

- **Estimates necessarily have large uncertainties**

**>> product-yield**

**>> market-value**

- **No monetary credit assumed for helium-3 production**

## **LICENSING AND SITING ISSUES**

- **Reactor licensing**

- >> **known to be a financial and procedural show-stopper**
- >> **for nuclear reactors**
- >> **especially in the United States**
- >> **and especially for non-traditional concepts.**

- **No applications for solution-reactor facilities in isotope production**

- >> **known to have been submitted for approval in United States**

- **Nuclear regulatory bodies have not developed regulations**

- >> **to facilitate solution reactors for commercial isotope production**

- **The two such reactors previously described were licensed**

- >> **by the U.S. Atomic Energy Commission**
- >> **but not as isotope-production facilities**



## **EXPEDITED LICENSING POTENTIAL**

- **Expedited licensing might be achievable**
  - >> **national-security priority in production of tritium**
  - >> **homeland-security considerations in production of He-3**
- **Might reduce construction costs and delays significantly**
  - >> **by siting on a government reservation**
  - >> **especially Oak Ridge or Savannah River**
  - >> **compared to an equivalent publically-sited plant**

## **INHERENT-JUSTIFICATION POTENTIAL**

- **Proposed liquid-fueled/liquid-cooled reactor**

- >> **appears to amply and quickly pay for itself**
- >> **provide near-term economic and national value**
- >> **more than enough to motivate government and commercial initiative**
- >> **especially in the United States**

- **Reactor could be fueled with uranium, thorium, and/or plutonium**

- >> **would satisfy multiple goals and professional interests**
- >> **including more efficient burnup, less byproduct waste**

## **DEVELOPMENT WORK REQUIRED**

- **Government and/or commercial development work needed**
- **Optimize production and separation**
  - >> **fission products, tritium, helium-3, and neptunium-237**
  - >> **financially self-supporting**
  - >> **providing public service national-security value**
  - >> **meet or exceed current national requirements for full cost-recovery**
- **Computations needed:**
  - >> **design-specific radiation-transport/nuclear-production**
- **Even if sited on a government reservation**
  - >> **licensing issues will again need to be addressed,**
  - >> **deserve to be expedited as much as possible**

## **SOME POINTS OF EMPHASIS**

**Here are some takeaway points of emphasis:**

- **NNSA and DOE**
  - >> **paying high cost for production**
  - >> **of tritium and helium-3**
- **Shortfall emerging in special radioisotope production**
  - >> **medical diagnostics and treatment**
  - >> **industrial research.**
- **Current medical-isotope production methods insufficient**
  - >> **production reactors around the world are aging**
  - >> **new reactors discouraged by proliferation concerns**
- **Small liquid-fueled reactor**
  - >> **could produce all the tritium**
  - >> **help resolve the medical-radioisotope availability**
  - >> **proven, extremely safe design**
  - >> **national-security and non-proliferation benefits**
- **Only one small specialized reactor needed**
  - >> **located at U.S. government site is needed**
  - >> **could be built and operated by private industry**

## **MORE POINTS TO EMPHASIZE**

- **Government facilities have had significant role**
  - >> **Oak Ridge National Laboratory reactor-development experience**
  - >> **Savannah River National Laboratory existing tritium processing**
- **Reactor products should readily compensate for the investment**
  - >> **tritium yield reduces federal government costs**
  - >> **rare radioisotopes sold commercially**
  - >> **steam, heat, electricity fungible byproduct**
- **Siting on a government reservation expedites/resolves**
  - >> **availability of the reactor**
  - >> **and its important radioisotope products**
  - >> **siting, licensing, and non-proliferation problems**

## **ADDITIONAL RELEVANT FACTORS**

- **At 100MWth, the reactor can be small, possibly modular**
- **Liquid-fueled reactors very efficient**
  - >> **in their use of low-enriched uranium as fuel**
  - >> **can consume natural uranium or thorium**
  - >> **fuel continuously recycled**
  - >> **good neutronic features**
- **Liquid-salt coolant/carrier**
  - >> **continuous circulation**
  - >> **enables very efficient on-line radioisotope extraction**
  - >> **allows removal of reactor poisons such as xenon**
- **Fluid fuels compared to solid fuels**
  - >> **much less radiation damage and thermal stress**
  - >> **consumed fuel replaced on-line during operation**
- **Operational features**
  - >> **atmospheric pressure**
  - >> **comparatively thin containment vessels**
  - >> **liquid fuel is continuously circulated**
  - >> **heat transferred at high temperature**
- **Enhanced safety characteristics are intrinsic to design**

## **MORE POINTS OF EMPHASIS**

- **Very safe reactor concept**
  - >> **liquid solution safely expands as temperature increases**
  - >> **reactivity control is intrinsic to design**
  - >> **passive safety valve from a solid meltable salt plug**
- **Concept indifferent to type of fuel**
  - >> **versatile fuel cycle**
  - >> **any combination of uranium, thorium, or plutonium**
- **Not a proliferation issue**
  - >> **especially if sited on a government reservation**
  - >> **especially if high-enriched uranium not needed**

## **SUMMARY**

**The molten-salt reactor of this concept:**

- **Fueled with thorium or uranium**
- **Would appear to produce timely and sufficient radioisotopes**
- **To meet or exceed current national requirements**
- **At the very least on a full cost-recovery basis, more likely at a profit**
- **With a potential commercial market product value**  
**>> of many billions of dollars per year**



## **APPENDIX**

### **ABSTRACT**

#### **Uranium-Thorium Liquid-Salt Reactor for Producing Valuable Radioisotopes**

Alexander DeVolpi (retired, Argonne National Laboratory)  
and

Ralph Moir (retired, Lawrence Livermore National Laboratory)

Valuable and indispensable radioisotopes – such as tritium, helium-3, molybdenum-99, and neptunium-237 – could be produced, separated, and extracted most efficiently and economically in a single, small liquid-salt reactor optimally situated on a government reservation.

Various international and national commissions have recognized looming shortfalls in some radioisotopes designated as essential. At present, nearly 80% of all nuclear-medicine procedures worldwide are derived from radioactive molybdenum-99, but there are no major producers in the United States. Much of the world's medical-isotope production is inefficiently carried out by irradiating uranium targets in aging specialized solid-fuel reactors. Production of tritium and helium-3 for national-security purposes has become increasingly expensive.

Much better sustained radioisotope production could be obtained from a liquid-salt reactor, an enterprising approach satisfying near-term high-priority goals for valuable and rare radioactive substances. One such small 100MWth reactor should suffice to meet domestic requirements for tritium, as well as international needs for medical radioisotopes, with a commercially profitable near-term return on investment.

The proposed reactor would be similar to the circulating molten-salt reactor originally developed at Oak Ridge National Laboratory. The isotope-production reactor's primary coolant would consist of F/Li/Be compounds that provide very good and relevant reactor-compatible properties, functioning with near-atmospheric pressure, reduced mechanical stress, simplified reactor design, and inherently safe operation. Mixed and circulated with the coolant would be criticality-sustaining fuel that could function with any of several fissile and fertile material combinations, primarily uranium and thorium.

Under typically constrained domestic circumstances — wherein national or commercial reactor funding, development, and construction options are limited — this particular concept offers near-term benefits while avoiding most shortcomings. Because of national-security considerations, the proposed liquid-salt radioisotope-production reactor likely would have to be federally endorsed, prioritized, and located on a government reservation.

## REFERENCES

- [1] *Medical Isotope Production Without Highly Enriched Uranium*, 2009, National Academies Press, 220pp.
- [2] D.A. Shea, D. Morgan, "The Helium-3 Shortage: Supply, Demand, and Options for Congress," Congressional Research Service (2010).
- [3] The Supply of Medical Radioisotopes: An Economic Study of the molybdenum-99 Supply Chain" Nuclear Energy Agency report 6967 (2010).
- [4] US Needs Med Isotope Production Capability" Union of Concerned Scientists, [www.ucsusa.org/news/press\\_release/us-needs-med-isotope-0182.html](http://www.ucsusa.org/news/press_release/us-needs-med-isotope-0182.html) (January 14, 2009).
- [5] U.S. NRC Backgrounder, "Tritium Production" (2005).
- [6] S. Willms, "Tritium Supply Considerations," Fusion Development Paths Workshop (2003).
- [7] Technology Assessment: Neutron detectors: Alternatives to using helium-3," GAO report (Nov. 2011).
- [8] Radioisotope Power Systems: An Imperative for Maintaining U.S. Leadership in Space Exploration," National Academies Press, 68pp (2009).
- [9] S. R. Green, *et al.*, "Pre-Conceptual Design of a Fluoride-Salt-Cooled Small Modular Advanced High-Temperature Reactor (SmAHTR)," Oak Ridge National Laboratory Report ORNL/TM-2010/199 (2010).
- [10] R. Hargraves and R. Moir, "Liquid Fuel Nuclear Reactors," *Forum on Physics & Society* (Jan 2011).
- [11] R. Thomas, *et al.*, "B&W Medical Isotope Production System: Codes and Modeling Strategy" (2011).
- [12] A. DeVolpi, "Lithium-Metal-Cooled Fission Reactor for Producing Radioactive Materials,," U.S. Patent and Trademark Office (March 2012).

## OTHER SOURCES

- >A description of the molten salt reactor on the thorium-uranium-233 cycle, from Reactor Physics and LPSC (Laboratoire de Physique Subatomique et de Cosmologie), Grenoble
- >Hoglund's home page, info and papers on molten salt reactors
- >Thorium fueled underground power plant based on molten salt technology, Ralph Moir and Edward Teller, Nuclear Technology 151 334-339 (2005)
- >C/C - SiC composites-white paper, Per Peterson
- >Nuclear Energy Research Initiative (NERI) Proposal 2002, Deep-Burn Molten-Salt Reactors, Moir et al
- >Cost of electricity from Molten Salt Reactors (MSR), Moir, Nuclear Technology 138 93-95 (2002)
- >Recommendations for a restart of molten salt reactor development, Moir, Energy Conversion & Management, 49 (2008) 1849-1858
- >Furukawa et al. ICENES 2007, Energy Conversion & Management, 49 (2008) 1832-1848
- >Jets and droplets compared to a moving slab of liquid for divertor cooling for a tokamak magnetic fusion energy reactor, (2008), R. Moir, Vallecitos Molten Salt Research Report No. 1, 51 pages
- >David LeBlanc's Too good to leave on the shelf, Mechanical Engineering, May 2010
- >Liquid fluoride thorium reactors, (2010), Hargraves and Moir, American Scientist, 98, May
- >Thinking nuclear? Think thorium.(2010), Kirk Sorensen. Machine Design, March 16
- >U232 nonproliferation features, (June 25, 2010) R. W. Moir. Vallecitos Molten Salt Research, Report No. 2, 17 pages
- >Liquid Fuel Nuclear Reactors, (January 2011) Robert Hargraves and Ralph Moir. APS Physics Forum on Physics & Society, Volume 40, Number 1
- >DOCTOR TELLER'S STRANGE LOVES (Web content)

## TABLE OF CONTENTS

BOTTOM LINE. ....	<a href="#"><u>2</u></a>
SCOPE OF PRESENTATION. ....	<a href="#"><u>3</u></a>
RADIOISOTOPE PRODUCTION AND APPLICATIONS. ....	<a href="#"><u>4</u></a>
MOLYBDENUM-99. ....	<a href="#"><u>5</u></a>
PRODUCTION OF RADIOISOTOPES. ....	<a href="#"><u>6</u></a>
NUCLEAR-WEAPON TRITIUM REQUIREMENTS . ....	<a href="#"><u>7</u></a>
HELIUM-3. ....	<a href="#"><u>8</u></a>
Plutonium-238. ....	<a href="#"><u>9</u></a>
NATIONAL-SECURITY AND NON-PROLIFERATION	
CONSTRAINTS . ....	<a href="#"><u>10</u></a>
SHORTFALLS	
IN MEDICAL-RADIOISOTOPE PRODUCTION. ....	<a href="#"><u>11</u></a>
LONGER-TERM RADIOISOTOPE SHORTFALLS. ....	<a href="#"><u>12</u></a>
PROPOSED LIQUID-SALT REACTOR	
FOR RADIOISOTOPE PRODUCTION. ....	<a href="#"><u>13</u></a>
GOVERNMENT SITING OF FACILITY. ....	<a href="#"><u>14</u></a>
ABOUT MOLTEN-SALT REACTORS. ....	<a href="#"><u>15</u></a>
MOLTEN-SALT BREEDER REACTOR EXPERIMENT. ....	<a href="#"><u>16</u></a>
SCHEMATIC OF MSRE. ....	<a href="#"><u>17</u></a>
<b>Figure 1.</b> Schematic diagram of a molten-salt reactor based on 1960's experiment at Oak Ridge	
.....	<a href="#"><u>17</u></a>
KEY FEATURES OF MSRE. ....	<a href="#"><u>18</u></a>
FLIBE. ....	<a href="#"><u>19</u></a>
AIRCRAFT REACTOR EXPERIMENT. ....	<a href="#"><u>20</u></a>
FEATURES OF NEW PROPOSAL	
FOR PRODUCTION OF VALUABLE RADIOISOTOPES. ....	<a href="#"><u>21</u></a>
ESTIMATING COMMERCIAL VALUE. ....	<a href="#"><u>22</u></a>
SUMMARY OF PRODUCTION EXPECTATIONS. ....	<a href="#"><u>23</u></a>
LICENSING AND SITING ISSUES. ....	<a href="#"><u>24</u></a>
EXPEDITED LICENSING POTENTIAL. ....	<a href="#"><u>25</u></a>
INHERENT-JUSTIFICATION POTENTIAL. ....	<a href="#"><u>26</u></a>
DEVELOPMENT WORK REQUIRED. ....	<a href="#"><u>27</u></a>
SOME POINTS OF EMPHASIS. ....	<a href="#"><u>28</u></a>
MORE POINTS TO EMPHASIZE	
.....	<a href="#"><u>29</u></a>
ADDITIONAL RELEVANT FACTORS. ....	<a href="#"><u>30</u></a>
MORE POINTS OF EMPHASIS. ....	<a href="#"><u>31</u></a>
SUMMARY. ....	<a href="#"><u>32</u></a>
APPENDIX. ....	<a href="#"><u>33</u></a>
ABSTRACT. ....	<a href="#"><u>33</u></a>
REFERENCES. ....	<a href="#"><u>34</u></a>
OTHER SOURCES. ....	<a href="#"><u>34</u></a>
TABLE OF CONTENTS. ....	<a href="#"><u>35</u></a>

### TEAC5 LSR Presentation

C:\Users\Nac\Documents\TEAC5 and Argonne\TEAC5 LSR Presentation.007.wpd