

Development of a Minichannel Compact Primary Heat Exchanger for a Molten Salt Reactor

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Abstract

The first Molten Salt Reactor (MSR) was designed and tested at Oak Ridge National Laboratory (ORNL) in the 1960's, but recent technological advancements now allow for new components, such as heat exchangers, to be created for the next generation of MSR's and molten salt-cooled reactors. The primary (fuel salt-to-secondary salt) heat exchanger (PHX) design is shown here to make dramatic improvements over traditional shell-and-tube heat exchangers when changed to a compact heat exchanger design. While this paper focuses on the application of compact heat exchangers on a Molten Salt Reactor, many of the analyses and results are similarly applicable to other fluid-to-fluid heat exchangers.

The heat exchanger design in this study seeks to find a middle-ground between shell-and-tube designs and new ultra-efficient, ultra-compact designs. Complex channel geometries and micro-scale dimensions in modern compact heat exchangers do not allow routine maintenance to be performed by standard procedures, so extended surfaces will be omitted and hydraulic diameters will be kept in the minichannel regime (minimum channel dimension between 200 μm and 3 mm) to allow for high-frequency eddy current inspection methods to be developed. High aspect ratio rectangular channel cross-sections are used. Various plant layouts of smaller heat exchanger banks in a "modular" design are introduced.

FLUENT was used within ANSYS Workbench to find optimized heat transfer and hydrodynamic performance. With similar boundary conditions to ORNL's Molten Salt Breeder Reactor's shell-and-tube design, the compact heat exchanger interest in this thesis will lessen volume requirements, lower fuel salt volume, and decrease material usage.

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I. Introduction

In order for a primary (fuel salt-to-inert secondary salt) heat exchanger be relevant in the future, a preliminary explanation of the need for nuclear power – by means of molten salt reactor technology – is first explained. Other changes and developments that would render this research highly relevant, including transition to thorium fuel and compact heat exchangers within the commercial nuclear power industry, are also explained and supported in this section. For reasons listed in the subsequent section of this report, a transition to the Molten Salt Reactor (MSR) for the commercial nuclear power industry could provide substantial improvements to several key issues with the current fleet of electrical power sources.

Current Energy Supply and Demand

The world's total energy usage is expected to increase substantially in the coming decades, and without significant changes in the sources from which the world draws its electricity, every current source presents some inadequacy. The United States Energy Information Administration estimates that the world's electrical demand will increase by nearly 50% from 2007 to 2035 in its International Energy Outlook 2010 (*IEO2010*) (U.S. Energy Information Administration, 2010). The Nuclear Energy Agency (NEA) expects an increase in electrical demand by a factor of 2.5 by 2050 (OECD Nuclear Energy Agency, 2008). While the actual rate at which population and electrical demand will increase is a point for debate, the notion that they *will indeed grow* is widely agreed upon. An increase in electrical demand will result from such a population growth, which will require new power sources that are not currently on-line.

As modest and relatively uniform increases are expected in economically stable countries belonging to the Organization for Economic Cooperation and Development (OECD)¹, massive growth is also expected from non-OECD nations. These developing, non-OECD nations are seeing rapid economic growth that will require quick improvements in infrastructure and electrical demand (OECD Nuclear Energy Agency, 2008). As these developing countries continue to grow and prosper to a level approaching OECD nations, the global electrical demand will grow accordingly. *IEO2010*'s Reference case, which assumes that no new laws are levied to change the manner in which electrical capacity is added to the global grid, estimates that non-OECD nations will see an 84 percent increase in electrical demand, while OECD countries should see a 14% increase in the same time span. Figure 1 shows the current and expected electrical demand, as explained in *IEO2010* (U.S. Energy Information Administration, 2010).

¹ Current OECD member countries (as of March 10, 2010) are the United States, Canada, Mexico, Austria, Belgium, Czech Republic, Denmark, Finland, France, Germany, Greece, Hungary, Iceland, Ireland, Italy, Luxembourg, the Netherlands, Norway, Poland, Portugal, Slovakia, Spain, Sweden, Switzerland, Turkey, the United Kingdom, Japan, South Korea, Australia, and New Zealand. Chile became a member on May 7, 2010, but its membership is not reflected in *IEO2010* (U.S. Energy Information Administration, 2010).

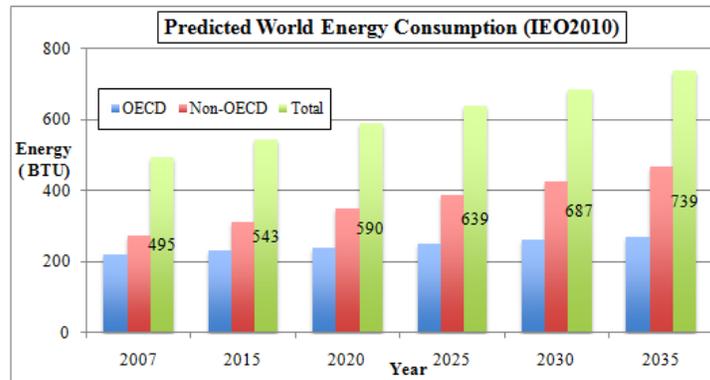


Figure 1

While OECD countries are expected to continue to look to nuclear energy to add to base-load power capacity, developing countries will most likely look to coal for cheaper, quicker energy. The development costs for nuclear power are most likely prohibitively high for a non-OECD country to consider, which could lead to an increase in coal power in the coming decades. The possibility of maintaining the amount of energy from coal power, let alone increasing its footprint, presents significant problems. The reasoning behind a movement away from coal power will be presented next.

Nuclear Power for the Future

The principal danger of fossil fuel power, and corresponding advantage of nuclear power, is the CO_2 emissions produced from burning coal. Global warming has been contributed mostly to carbon emissions from coal power by several sources (U.S. Department of Energy and Environmental Protection Agency, 2000). The Intergovernmental Panel on Climate Change (IPCC) has claimed that the emissions level from 2005 are twice the level of what is reasonably safe, so major reductions in emissions must be made in both the short-term and long-term (Solomon, Qin, & Manning, 2007). The only means by which to halve carbon and greenhouse gas emissions is to lessen the share coal-fired plants hold on the base-load power of the United States. Nuclear power is the only *proven* commercial electricity source that can shoulder such a large load. Without the presence of nuclear power to supply approximately 16% of the world's current electricity supply, nearly 2.5 billion more tons of CO_2 per year would be emitted via coal-fired plants (World Nuclear Association, 2010).

Global warming from carbon and greenhouse emissions is not the only negative outcome of coal power, especially when compared to the nuclear power industry. SO_x , NO_x , and fine particulates released from the burning of fossil fuels have also been heavily linked to severe adverse health effects. The “fly ash” emitted from coal power is a danger that nuclear power does not present, as any radioactive byproducts are contained within fuel rods in current nuclear reactors.

Energy security is an ongoing issue with fossil fuels due to the scarcity of the over-used, less abundant resources; whereas, the resources of fuel for nuclear power have been estimated to be sufficient for fifty- to several hundred-years. The amount of nuclear fuel needed is also drastically lower than that of coal-fired plants, which is seen when one ton of uranium produces the same amount of energy as roughly 10,000-16,000 tons of coal (OECD Nuclear Energy Agency, 2008). Transportation costs for providing these larger amounts of fuel are yet another increased contribution from coal power on carbon emissions.

Nuclear energy provides the best and most reliable source of power that is virtually carbon-free. Figure 2 demonstrates compares nuclear power to other industries, in regards to the amount of carbon emissions resulting from each method (World Nuclear Association, 2010).

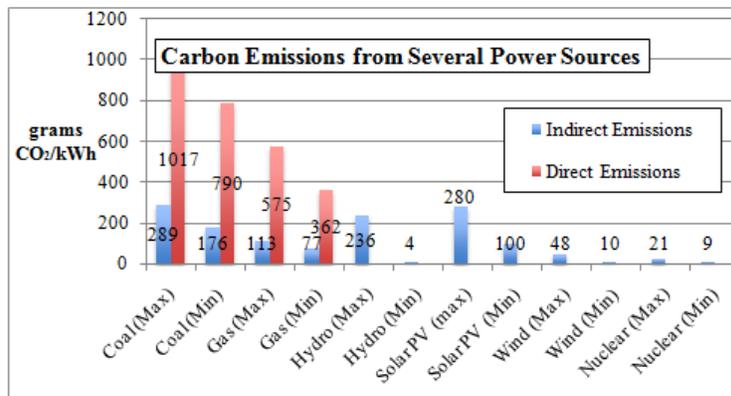


Figure 2

While nuclear power and coal are the chief competitors for base-load power in the United States, renewable energy sources have been seen as a possible replacement for both. Wind and solar energy have become very popular with environmentalists recently and can be contributors in the future, but they are simply not a realistic option to provide a significant amount of electricity. Capacity factors, the percentage of time during which 100% power is able to be produced by a given electrical production method, are well below 50% for alternative energy sources (~20-40% for wind and ~12-15% for solar). Nuclear power operates with capacity factors well above 90%, with even larger values found during years without re-fueling. The overall heat exchange systems and steam cycle used in nuclear and coal power is also much more efficient than the turbines used in wind and the electrical conversion used in solar. Due to these technical issues, as well as an unproven record of successful use on the necessary scale, depending on renewables for a significant amount of energy in the near future would be problematic. These “renewable,” or “green,” energy sources have received a great deal of political support, but are not commercially viable on a scale that will provide noticeable differences in carbon emissions world-wide.

The use of nuclear energy has been a polarizing debate since the accident at Three Mile Island in 1979, but the technology used on new Generation III Light Water Reactors (LWRs) is much more advanced than the designs involved in the highly publicized accidents. In addition to more stringent regulations and design standards throughout the industry, nuclear reactor operators are now rigorously tested and licensed. As a highly-refined technology with passive and redundant safety barriers, the nuclear industry must be relied upon to add dependable, emission-free electrical power in the future. Nuclear power holds the potential to be the safest, cleanest, and most efficient power source in the world.

The Light Water Reactor (LWR) design that is currently used in all commercial reactors in the USA, now about 40 years old, still has plenty of room for improvement in future generations of nuclear plants. Inefficient thermal-electrical conversion systems are robbing LWRs of electrical power due to the modest Carnot efficiencies seen in the Rankine cycle. With water also used as the working fluid in the LWR, the system must be operated at very high pressures to avoid boiling. High-pressure systems are much more susceptible to rupture or stress damage, which presents an obvious disadvantage. The issue of spent nuclear fuel, often referred to with the misnomer “nuclear waste,” requires significant storage and protection. The accumulation of the spent fuel leads to storage costs and security costs, mostly due to the possible proliferation path present due to the presence of plutonium after burning uranium. In addition to the LWR technology needing some improvement, the uranium fuel cycle used in today’s LWRs may not be the best option for the future of nuclear power. The advantages of moving away from uranium fuel will be presented in the subsequent section of this report, and the advantages of pursuing advanced reactor designs will be explained thereafter.

Thorium as a Nuclear Fuel

The concept of using thorium as a nuclear fuel has existed since the 1950’s, when Alvin Weinberg, the former Director of Oak Ridge National Laboratory (ORNL), was a proponent of this new nuclear fuel as the technology of the future. Weinberg thought of thorium as a means for affordable, safe, and efficient energy that can actually *create* its own fuel as it operates and produces power. He proposed that it be used for civilian nuclear power for creating electricity, but the plutonium-producing uranium LWRs were eventually chosen due to their greater operational experience and ability to provide a means for the production of plutonium (Weinberg, 1997). Thorium has garnered new interest with those followers favoring its ability to provide nearly carbon-free, nearly waste-free power (Martin, 2009).

Thorium has several inherent advantages over uranium, some of which are (International Atomic Energy Agency, 2005), (Raghd & Tsoukalas, 2010):

- *Abundance.* Thorium is roughly 4 times more abundant than uranium in the Earth’s crust. Vast thorium reserves can also be found on the moon and Mars, in case the Earth’s

supplies have been exhausted (Raghd & Tsoukalas, 2010). Thorium ore does not naturally contain any isotopes other than the “fissile” isotope Th^{232} , so all thorium is useful as nuclear fuel. This is not the case with uranium, as explained in the next point.

- *No enrichment necessary.* Uranium requires a relatively expensive, difficult, and dangerous enrichment process of natural uranium to increase the amount of U^{235} from its natural amount (~0.7 weight %) to an amount desired for use in reactors (~3-5 weight %). The enrichment process is a potentially dangerous proliferation path for low-enriched uranium, which can then be further enriched to make weapons-grade material. Thorium would completely eliminate the need for enrichment and would close one proliferation path that concerns some critics.
- *Inherent proliferation resistance.* Thorium forms U^{232} through (n, 2n) reactions, which is a very strong gamma emitter. Although pure U^{233} is acceptable as weapons material, the gamma radiation from U^{232} is a deterrent for bomb-makers. Even though the use of thorium does not completely eliminate all proliferation risk, thorium provides a less-preferred path.
- *Thermal breeding.* U^{233} has a thermal reproduction factor greater than 2, which makes thermal breeding possible. Uranium-235 is not able to perform breeding, and plutonium can only do so in the fast neutron spectrum. Breeding in the fast neutron spectrum remains highly unproven and more volatile than its thermal counterpart which has been performed at ORNL and Shippingport, PA (International Atomic Energy Agency, 2005). Thorium also has a higher neutron absorption cross-section than uranium, so fission occurs more easily and efficiently.
- *Lower spent fuel volume.* Thorium produces much less Minor Actinides and plutonium than the uranium fuel cycle, which was previously chosen over thorium *because* it produced plutonium (Weinberg, 1997). Spent fuel will still accumulate in solid-fueled reactors utilizing thorium but at much smaller volumes. Liquid-fueled reactors could produce even less spent fuel, when used with thorium.
- *Limited radiotoxicity.* Due to a lower amount of Minor Actinides produced in the thorium fuel cycle, the long-lived radiotoxicity is significantly reduced. Fewer isotopes with shorter half-lives are produced by thorium, so the lower volume of spent fuel will also be less radioactive.
- *Chemical advantages.* Thorium is known to be more chemically stable in oxide form than uranium with better resistance to radiation damage as well. In the oxide form, thorium also has a lower coefficient of thermal expansion, higher thermal conductivity, and less oxidation inertia.
- *Enhanced safety.* Better temperature coefficient, void reactivity coefficient, and lower excessive reactivity are all inherent safety features that are improved by using the thorium fuel cycle. These features of thorium will inherently provide for enhanced passive controls.

- *Versatility.* Thorium is capable of thermal breeding, as explained previously in this section. This was demonstrated at the Shippingport reactor, where thorium was introduced into a used PWR core. Thorium can be used alone or with uranium and/or plutonium. Thorium can be used in breeder reactors (which breed new fuel as they produce power), “burner” reactors (which burn plutonium from weapons or spent fuel as they produce power), “converter” reactors, and even fusion reactors (LeBlanc, 2010). Thorium has also been used to flatten power distributions across uranium-based reactor cores (Beedie, 2007). The solubility of thorium in molten fluoride salts also intrinsically ties it to use in molten salt reactors.
- *Experience.* Thorium has been successfully used in the Shippingport reactor and High-Temperature Gas-cooled Reactor (International Atomic Energy Agency, 2005). While it does not boast the decades of operational experience available with uranium, thorium is far from unknown.

Although thorium is still unproven compared to the uranium fuel cycle in the United States, its use is relatively extensive and has been successfully demonstrated throughout the world in several different projects since the 1960’s. Table 1 shows a list of all reactors tested with thorium fuel, where the most impact has been seen in countries other than the United States (International Atomic Energy Agency, 2005):

Table 1: Reactors using thorium worldwide since the 1960's (International Atomic Energy Agency, 2005).

| Name | Country | Type | Power | Fuel | Operation Period |
|-------------------------------|--------------------|---|-----------------------------------|--------------------|--|
| AVR | Germany | HTGR Experimental (Pebble bed reactor) | 15 MW(e) | Th-U | 1967-1988 |
| THTR | Germany | HTGR Power (Pebble Type) | 300 MW(e) | Th-U | 1985-1989 |
| Lingen | Germany | BWR Irradiation-testing | 60 MW(e) | Th-Pu | Terminated in 1973 |
| Dragon | UK OECD/Euratom | HTGR Experimental (Pin-in-Block Design) | 20 MW(t) | Th-U | 1966-1973 |
| Peach Bottom | USA | HTGR Experimental (Prismatic Block) | 40 MW(e) | Th-U | 1966-1972 |
| Fort St Vrain | USA | HTGR Power (Prismatic Block) | 330 MW(e) | U-233 | 1976-1989 |
| MSRE | USA | MSBR | 7.5 MW(t) | Th-U | 1964-1969 |
| Borax IV and Elk River | USA | BWRs | 2.4 MW(e), 24 MW(e) | Th-U | 1963-1968 |
| Shippingport and Indian Point | USA | LWBR PWR | 100 MW(e), 285 MW(e) | Th+HEU | 1977-1982, 1962-1980 |
| SUSPOP/KSTR KEMA | Netherlands | Aqueous Homogeneous Suspension | 1 MW(t) | Th-U | 1974-1977 |
| NRU and NRX | Canada | MTR | | Th-U | Irradiation-testing of few fuel elements |
| KAMINI/CIRUS/DHRUVA | India | MTR Thermal | 30 kW(t)/40 MW(t)/100 MW(t) | Al-U- 233/Th/Th | All in operation |
| KAPS 1/2, KGS 1/2, RAPS 2/3/4 | India | PHWR | 220 MW(e) | Th | Continuing in all new PHWRs |
| FBTR | India | LMFBR | 40 MW(t) | Th | In operation |

Thorium is an intriguing option for the future of nuclear power, and it appears to be the easiest way to appease the critics of nuclear power that cite “nuclear waste” as the key issue with the technology. While the United States is still only showing limited interest in the thorium fuel cycle, other countries are beginning to take notice of its multitude of advantages over uranium.

Thorium is at the center of the 3-stage plan India has put in place to strengthen its domestic power capabilities (International Atomic Energy Agency, 2005). India has scarce uranium reserves but has a wealth of thorium to utilize in its nuclear reactors. Indians have developed an interconnected system of reactors that all utilize thorium in different forms and concentrations to provide a self-sustaining cycle for the future. The United States is believed to have even larger thorium deposits than India, according to recent findings (Raghd & Tsoukalas, 2010).

Thorium may be on its way to breaking through in the United States as well, as the Lightbridge Corporation is designing fuel assemblies similar to those at Shippingport to create low-waste nuclear power (Lightbridge). The Thorium Energy Alliance and members of EnergyFromThorium.com are also civilian groups interested in bringing thorium to the forefront of the nuclear industry.

Thorium is intrinsically tied to the Molten Salt Reactor (MSR), since it was successfully used in the Molten Salt Reactor Experiment (MSRE) at Oak Ridge National Laboratory (ORNL) in the 1960's. Although it has mostly been tested as a solid thorium dioxide (ThO_2), thorium may be best suited for use in a liquid-fueled reactor such as the MSR.

Molten Salt Reactors

Molten Salt Reactors are an innovative branch of nuclear reactors that allow nuclear fuel (thorium, uranium, and/or plutonium) to be dissolved into liquid salts, where the salt acts as the working fluid to transport heat. Nuclear fission heats the primary salt, a molten fluoride compound, which is pumped to a primary heat exchanger to recover the added energy. The primary heat exchanger transfers the heat created in the fuel salt to an inert secondary salt. The secondary salt then flows to an intermediate heat exchanger, where the heat is transferred to steam, super-critical CO_2 , or some other working fluid for the electricity-generating turbine (Bettis, et al., 1967). The liquid fuel used in MSRs is in stark contrast to solid oxide fuel rods used in LWRs, and has many advantages that the solid-fueled reactors cannot offer.

Molten Salt Reactors have many advantages, with several of the most influential explained here (Raghd & Tsoukalas, 2010), (LeBlanc, 2010):

- *Compatibility with thorium.* Thorium is soluble in fluoride salts, which are often the working fluid in MSRs. The Molten Salt Reactor Experiment used thorium successfully during its operation. Several designs included a homogeneous mixture of UF_4 and ThF_4 , while other designs maintained a separate two-fluid design with thorium as a “blanket” salt. The ability of thorium to achieve thermal breeding, thus creating its own fuel while it creates power, could be further improved by the ability of the MSR to perform on-line reprocessing.
- *Liquid fuel.* Reactor “meltdown” is impossible because the fuel is already in liquid form. Fuel can be drained into separate “dump tanks” to safely and passively stop the fission chain reactions in a safe manner, as necessary.
- *Lower fission product accumulation.* Many fission products form stable fluorides and remain within the salt during leaks or ruptures. Others can be bubbled out (noble gases) or plate out on metal surfaces (noble metals), each providing simple means for capture. Liquid fuels can have their neutronics performance improved by performing on-line fuel reprocessing.

- *Low-pressure operation.* MSR can operate near zero-pressure due to the extremely high boiling point and low vapor pressure of molten salts. This allows for thinner-walled components than LWRs, which operate at roughly 2250 psi, throughout the reactor system. The risk of pipe rupture is significantly reduced in such a low-pressure system.
- *Higher burnup.* The accumulation of fission products in solid fuel assemblies causes fuel rods to be removed from the core before the majority of the energy is extracted. The fuel rods still contain a great deal of radioactivity and energy, thus making them more difficult to handle. MSR flow a homogeneous mixture of fuel salt through the core, which constantly mixes to allow for an even, thorough burn of the fuel. The removal fission products, which leach neutrons and hurt neutronics performance, can be performed continuously as the reactor is operating.
- *Better fuel utilization.* Higher burnup of the fuel leads to a dramatic decrease in transuranic wastes, when compared to the standard uranium fuel cycle. This requires less waste storage, and the waste that is left behind is not nearly as harmful as spent fuel from a uranium plant (based on the constituent radioactive makeup of spent thorium fuel).
- *Xenon is continuously removed.* Xenon is “bubbled off” and removed automatically in the primary system of MSR, which simplifies the operation of the plant at startup and shutdown. No excess reactivity is needed during fueling to deal with power decreases and “dead time” from xenon, as solid-fueled reactors must deal with.
- *Molten salts have excellent heat transfer characteristics.* Molten salts carry heat very efficiently, but are hardly more difficult to pump than water. Higher volumetric heat capacity leads to less salt inventory and smaller components (heat exchangers, pumps, pipes, etc.). Some advanced reactors may begin using molten salts as secondary coolants due to their better heat transfer characteristics (Figley, 2009).
- *Low fissile load required.* Fissile loads of only $\sim 800\text{kg}/\text{GW}_e$ are needed for break-even operation of certain MSR. This is a tremendous improvement from the larger amounts for current LWRs, and even more so for fast reactors. Smaller components and smaller fuel volumes result from more efficient heat creation and transfer.
- *Enhanced safety.* Temperature coefficients and void reactivity coefficients are more negative in MSR than in LWRs. Lower excessive reactivity, as a result of constant xenon removal, adds yet another safety advantage. Numerous passive control systems, which will be explained later, will add another level of enhanced safety.
- *On-line refueling and reprocessing.* Reprocessing can allow for improved neutron economy, if desired. The ability to perform reprocessing, refueling, and repairing on-line allows for higher capacity factors.
- *Passive control systems.* Control rods can be used, but are not required. A “freeze valve”, located below the reactor, can also be used as a passive safety system. The freeze valve remains solid until the high-temperature fuel salt is allowed to flow toward it, in which case it melts and allows fuel to drain to several separate “dump tanks”. Dump tanks are located below the reactor to stop the fission chain reaction and allow for cooling of the

decay heat still being produced in the fuel. MSR's can also achieve automatic load following for major fluctuations in reactor power to provide greater stability.

- *High-temperature operation.* Molten salts have very high boiling temperatures, so MSR's can be safely operated at temperatures approaching 1000°C, materials issues notwithstanding. Higher temperatures will improve plant efficiency and could also pair with hydrogen production and other high-temperature process streams.
- *Scalable.* Full-sized plants can be built to compete with current 1 GW LWR's. Molten Salt Reactors can easily be scaled down to provide modular power sources, as well.
- *Experience.* Experimental Molten Salt Reactors have been designed, built, and operated at Oak Ridge National Laboratory during the past 50 years. A wealth of knowledge and experience was gained with the studies performed at Oak Ridge. Several new advanced reactor designs must still be completely developed from scratch, but MSR's already have a base upon which to build.
- *Versatility.* Different salt and fuel variations allow for different functions of MSR's. Fluorides can be used for thermal breeders or transuranic burners. Chlorides can be used for fast reactors. Molten salts can also be used solely as the secondary coolant to utilize its superior heat transfer characteristics (Figley, 2009).

The novel design of the MSR does not come without challenges, as operating experience from Light Water Reactors does not apply to Molten Salt Reactors. The components of MSR's must be designed almost entirely independent of current components due to the unique constraints with which the system operates. The specific challenges encountered with molten salts will be outlined later in this section.

Molten Salt Reactors are a revolutionary nuclear plant design that differs greatly from Light Water Reactors in many aspects. Whereas water is the working fluid and solid fuel elements are where fission takes place in LWR's, the molten fluoride salt is both the working fluid and the source of fission reactions in MSR's. Molten salts operate at very high temperatures and very low pressures, which is possible due to the advantageous physical properties of molten salts, and a vital set of advantages is seen for those reasons.

In a MSR, the fuel salt flows through a graphite moderator in the reactor core, where it reaches criticality. Heat is generated through fission in the core, where the graphite moderates neutrons to maintain criticality in the core. The fuel salt then flows out of the core to the primary heat exchanger, which extracts heat from the fuel salt to the coolant salt. The coolant salt then heats a tertiary working fluid, which is now fully removed from any radiation effects, which eventually is used to create electricity. This function allows the secondary side of the plant to be non-radioactive, much like a Pressurized Water Reactor (PWR).

Downstream of the primary heat exchanger, several design possibilities exist for the Balance of Plant (BOP). Secondary heat exchangers can transfer heat to a steam cycle or a variety of gas

cycles, with each case requiring a separate set of design requirements. ORNL designs from the 1960's used traditional boilers in a steam cycle (Bettis, et al., 1967), while modern concepts plan for Brayton gas cycles (Figley, 2009).

A fraction of the salt leaving the core, instead of heading toward the primary heat exchanger, can be bypassed to a chemical processing plant. The chemical processing step is optional but provides advantages in neutron economy and fuel burnup. This function will most likely be added as an additional feature after preliminary MSR designs have been validated on a large scale.

Molten Salt Reactors exist in two varieties: single-fluid and two-fluid. In a single-fluid design, the reactor core is a homogeneous area of fuel salt and graphite moderator. Fuel salt flows through the reactor, increases in temperature, and transfers heat to a secondary fluid in the Primary Heat Exchanger (PHX).

Figure 3 depicts the simple process followed in a single-fluid MSR design, which will not be the focus of this study.

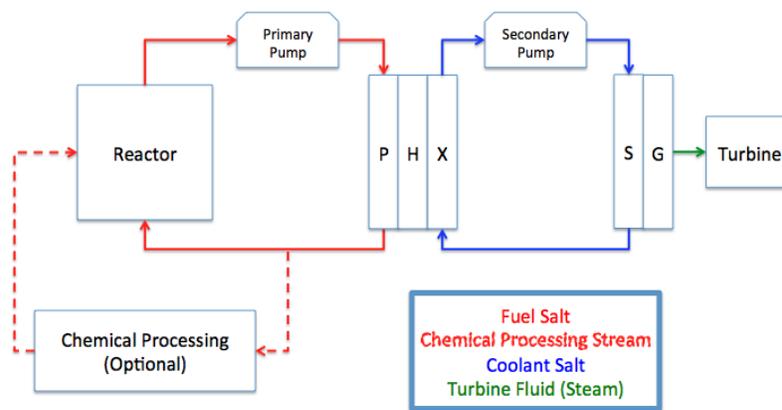


Figure 3: Schematic of single-fluid MSR design.

Single-fluid MSRs were able to provide a thermal breeding ratio barely in excess of 1 during their initial design studies. While anything in excess of 1 is impressive because it is creating fuel as it operates, interest was gained in attaining breeding ratios significantly higher than 1. This desire fueled interest in the two-fluid design, which was expected to out-perform such a design in terms of neutronics.

The flow process and piping of a single-fluid design is substantially simplified compared to the two-fluid design, but significant advantages in neutronics can be seen by creating two separation regions in the core: a graphite moderator region for the fuel salt and an un-moderated blanket region for a separate salt. The segregated fluid regions provide additional moderators for neutrons that may escape in a single-fluid design and allow for the possibility of substantial

thermal breeding, which was not viable in a single-fluid design. Fuel processing is not necessarily required for a two-fluid design as it is for a single-fluid design that hopes to achieve a breeding ratio in excess of one. With a focus on achieving the highest breeding ratio possible, momentum shifted to the two-fluid design at ORNL.

A two-fluid design, in contrast to the single-fluid design shown in Figure 3, has separate fluid streams for the fuel salt and blanket salt. The fuel salt, a uranium-containing FLiBe compound, differs from the blanket salt, a thorium-containing FLiBe compound, in a two-fluid design. In order to improve the neutron economy of the reactor design, the blanket regions acts as a reflector and region to capture any neutrons that would have escaped in a single-fluid design. The heat produced in this region is lower than in the fuel salt region, but its inclusion is necessary to cause as much fission as possible. The fuel salt follows a closed path through the graphite moderator, into the fuel salt pump, down through the Primary Heat Exchanger (PHX), and back to the moderator region of the reactor core. The blanket salt follows a closed path through the blanket region, through the blanket pump, into the Blanket Heat Exchanger (BHX), and back into the blanket region of the reactor core. The secondary coolant salt travelled upward through the PHX, transferring heat from the fuel salt to the coolant salt, which was then passed to the BHX, where it picked up the additional heat before moving to the BOP. Figure 4 shows a diagram of a two-fluid MSR from ORNL in 1968 (Robertson, Smith, Briggs, & Bettis, 1968).

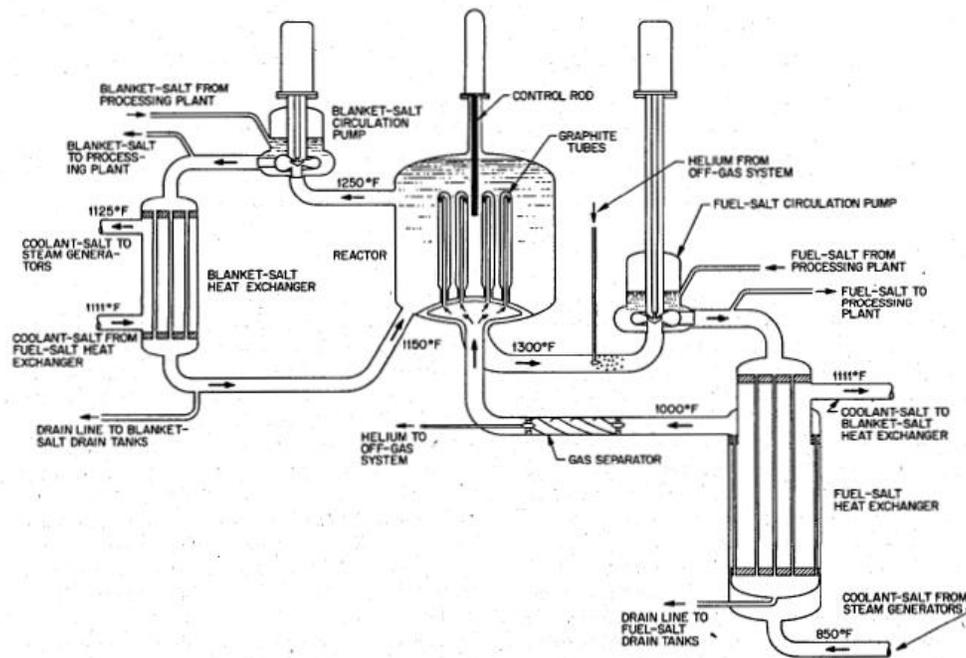


Figure 4: Schematic of two-fluid MSR design from ORNL, courtesy of Oak Ridge National Laboratory (Bettis, et al., 1967).

Progress evolved rapidly at ORNL in the 1960's when the two-fluid design took control due to its ability to achieve thermal breeding ratios over 1. As the research developed and matured

while being backed by Director Alvin Weinberg, the MSR concept later fell out of favor in the early-1970's in favor of the Liquid Metal Fast Breeder Reactor (LMFBR) (LeBlanc, 2010). The technology remained dormant for nearly 40 years, but recent concerns with LWR designs and interest in using thorium has led India and China to begin developing MSR designs (Martin, 2011).

Modern domestic design concepts, such as the Liquid Fluoride Thorium Reactor (LFTR), have begun to garner interest in the MSR technology once again in the United States, but there are no completed designs with the level of maturity associated with ORNL's MSBR concept. Significant funding and research efforts will be required to bring this promising technology to fruition.

In the time since Alvin Weinberg's final MSR concept was designed, few innovations have been made in MSR designs, but the technologies around which the plant was built have grown dramatically. Advances in materials, computing, chemistry, and fabrication technologies would now render much of the MSBR design obsolete. The MSR concept that was submitted and accepted as a Generation-IV concept recently is relatively unchanged from the plant designed to be the MSBR several decades ago, but updates still need to be made throughout the design.

The materials-related issues with MSRs were studied heavily during the 1960's but were left somewhat unresolved with the termination of the MSBR project. As material issues with MSRs were among the reasons the AEC chose to move forward with LMFBRs, nearly all of those issues were completely alleviated by the time the Molten Salt Reactor Program ceased to perform research at ORNL (MacPherson, 1985). Great achievements were made to identify, design, and invent Hastelloy N, but it still has yet to be proven over the large time frame needed for reactor components. Modern advanced materials and fabrication techniques should allow for further improvements in performance.

Licensing will be another issue of difficulty for the MSR, as the NRC has never approved a commercial liquid-fueled design. Claims of "inherent safety" will obviously not be sufficient in a final design, but the passive safety systems used on MSRs should show an improvement over the already impressive LWR track record of safety.

The most important aspect of updating the design is using improved components that have the advantage of better technologies in their design, analysis, and construction. The use of Finite Element Analysis (FEA) in the design of components and core behavior should allow for much more concise results with an added level of comfort in their correctness (Figley, 2009). When compared to the rudimentary techniques used in the original MSRE and MSBR design, modern components should perform significantly better, be safer, and allow for more aggressive safety factors.

Heat exchangers, which are a vital component in the overall efficiency of any plant, have made rapid developments since the 1960's. Standard shell-and-tube heat exchangers are currently used

throughout the nuclear industry, and have been since the 1960's, but modern advancements may provide room for significant improvements. This study focuses on the Primary Heat Exchanger, which is subjected to extreme environmental conditions that may or may not be suitable for vast improvements.

Primary Heat Exchanger

Based on encouraging results from the MSRE, the Molten Salt Breeder Reactor (MSBR) was designed to be the first full-scale, commercial nuclear power plant utilizing molten salt liquid fuels (Kasten, Bettis, & Robertson, 1966). Shell-and-tube heat exchangers were the preferred heat exchangers at the time the MSBR was designed, so all heat exchangers in the design were of the shell-and-tube variety. Two types of molten-salt-related heat exchangers, primary and blanket, were required for the primary system of a two-fluid MSBR. The primary heat exchanger, the focus of this study, will be explained and studied in further detail.

The function of the primary heat exchanger is to recuperate heat from the fuel salt carrying the fissile load to an inert secondary salt. Several secondary heat exchangers are required to convert a heated secondary salt to water, which is converted to steam in a "boiler." The BHX is subjected to very similar conditions as the PHX, and the concept studied in this research will also be relevant in updating the BHX design where required. Very high efficiency heat conversion is desired for the primary heat exchanger, as any heat that is not transferred to the secondary salt is a direct detriment to overall plant efficiency. The MSBR heat exchanger was designed as a shell-and-tube heat exchanger two-pass vertical exchanger with disk and doughnut baffles (Bettis, et al., 1967). Four such exchangers were necessary for the four separate heat-exchange loops employed in the MSBR design. Each heat exchanger was able to remove 528.5 MW of heat from the core. Figure 5 shows a simplistic sketch of the shell-and-tube PHX taken from Figure 4, with its imposed boundary conditions, its required header locations, and its general shape more clearly shown.

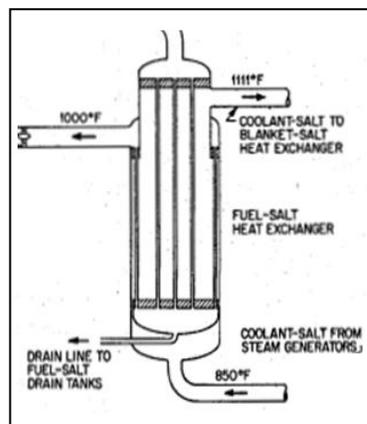


Figure 5: ORNL shell-and-tube heat exchanger's boundary conditions, courtesy of Oak Ridge National Laboratory.

The primary heat exchanger of a MSR is subjected to a unique set of conditions that brings forth several design challenges not encountered in standard heat exchangers. The somewhat corrosive molten salts, especially at temperatures in excess of 700°C, require specialized materials throughout the system to avoid corrosion, radiation damage, and adverse high-temperature effects such as creep. For these reasons, Hastelloy N was created and tested for use in Molten Salt Reactors (Grimes, 1967). Hastelloy N comprised the entire construction of the primary heat exchanger of the MSBR, each of which was roughly 6.5 feet in diameter and 22 feet in height (Bettis, et al., 1967). Hastelloy N is a material with limited commercial uses, so it is expected to be very expensive compared to more common stainless steel alloys. Four heat exchangers of the previously prescribed dimensions would be a sizeable capital cost for use with any material but are especially troubling considering the exorbitant cost of Hastelloy N. For this reason, any manner in which the volume of the heat exchanger could be significantly reduced would be a step in the direction of making molten salt reactors economically competitive. A reduction in the amount of material used in that volume is also a vital, and separate, calculation that must be made.

The progress made at ORNL on the shell-and-tube PHX design will be explained in detail later in this report. Key design parameters, sketches, and site layout plans were included in the ORNL documentation and will provide a means for comparing the ORNL design to the design presented in this study. The amount of research and development performed in the 1960's and 1970's at ORNL conveys the serious interest that MSRs invoked over the decade-long period of their incubation. The considerations and breakthroughs made in that time period can be enhanced by utilizing modern compact heat exchangers to more efficiently transfer the requisite heat load.

Compact Heat Exchangers

Shell-and-tube heat exchangers are still the preferred concept in LWRs and many other applications today, but the compromised efficiency seen with their use may not outweigh their relatively more advanced operational experience in future designs. Compact heat exchangers, which use sophisticated fabrication techniques to form extremely small (well below 1mm in diameter) micro-channels, allow for more complete heat transfer in much smaller volumes (Ashman & Kandlikar, 2006). Advanced fabrication methods, such as the chemical etching process used for the Printed Circuit Heat Exchanger (PCHE), allow for small-scale fabrication of such complex heat exchanger geometries in a manner similar to how computer circuits are fabricated (Kanaris, Mouza, & Paras, 2004). The use of very small micro-channels allows for similar heat transfer areas to be accommodated in much smaller total volumes, while using less solid material in the process.

Modern compact heat exchangers can provide compactness, a measure of the ratio of surface area-to-volume of a heat exchanger, approaching $2500 \text{ m}^2 / \text{m}^3$ for the most advanced designs. When compared to a compactness of only approximately $43 \text{ m}^2 / \text{m}^3$ for the shell-and tube designed for the MSBR (6.5 feet diameter and 22 feet height, while accommodating a heat transfer area of nearly 900 m^2) (Kasten, Bettis, & Robertson, 1966), compact heat exchangers can provide significant reductions in volume and material usage. With tightly packed channels making adjacent streams able to effectively translate temperatures across their boundaries, approach temperatures (the difference between the outlet temperature of one fluid stream and the inlet temperature of the opposing fluid stream at their common header location) closer to 1°C are possible with compact heat exchangers. Shell-and-tube heat exchangers are often closer to 12°C in approach temperature, another demonstration of their inferior performance (Kandlikar, Garimella, Li, Colin, & King, 2006). Compact heat exchangers are an intriguing technology that could provide MSRs, and nuclear reactors in general, with a means to reduce capital costs.

Compact Heat Exchanger History

The first sign of movement to relatively “compact” heat exchangers was seen in the automotive industry in the early 1900’s, when an improved manner for removing heat from engines with minimal material (weight) was sought. Whereas heat was previously removed from engines by boiling a pool of water and releasing it into the air in an unsophisticated convection process, heat exchangers were seen as an inexpensive way to improve engine performance (Shah & Sekulic, 2003).

Automakers began to use closed volumes of liquid arranged in “serpentine” pipes to increase the heat transfer area and remove the unpredictable boiling process. From this starting point, engineers have continued to find new ways to increase the performance of automotive radiators and heat exchangers in countless other industries. With the limits of standard serpentine arrangements being reached, extended surfaces were then added to more efficiently remove heat, and pipe geometries were altered to provide improved performance. The automotive industry, however, had little need for significantly enhanced performance past the designs of the early 1900’s. Several other industries provided a great deal of room for improvement and proved to be the launching point for the oft-researched compact heat exchanger field (Shah, McDonald, & Howard, 1980).

Several years of technology advancement on the aircraft internal combustion engine led to a much better understanding of the physics and abilities of extended surfaces as heat transfer enhancers. The first compact heat exchanger that resembled anything seen today was made for the after-cooler of the jet engine during World War II, with enough success and interest shown to interest the United States Navy Bureau of Ships to begin testing compact heat exchangers on their gas turbines (Shah, McDonald, & Howard, 1980).

The Navy's work in the 1940's was extended to research at Stanford University throughout the next two decades, and that group made several of the most fundamental achievements in compact heat exchanger design. Culminating with the first publication of works by A.L. London, who later teamed with W.M. Kays to lead the compact heat exchanger field with their fundamental texts, the groundwork was laid for the massive improvements that have come with compact heat exchangers in the decades since work began. The work by the Navy and Stanford University, coupled with improved computing abilities, helped draw interest from several industries that could benefit from the advantages offered by more sophisticated heat exchanger design.

Heat exchanger designs continued to expand and develop through the 1960's and more industries gained interest in using compact heat exchangers to increase efficiency. Compact heat exchanger technologies spread from the automotive industry to the air conditioning industry, several manufacturing industries, and even to the magnetic railway industry (Shah, McDonald, & Howard, 1980).

The 1980's brought an increase in computing power and function that led compact heat exchanger research away from experimental solutions in favor of Computational Fluid Dynamics (CFD). CFD provided low-cost means to simulate the physics occurring inside heat exchangers relatively accurately, without the need to construct and test physical apparatuses before a final design was developed. It took several years, and decades in some cases, for the computing power to be able to accurately predict the physics in a timely fashion for the incredibly small geometries in compact heat exchangers. That breakthrough allowed for significant advancements in the industry. Optimization processes were more easily obtained, and more complex geometries could be tested with the use of CFD.

In the time since CFD became prevalent in heat exchanger design, several systems of commercial codes have been developed to allow for rapid testing and simulations. The programs continue to become more accurate, and as computers continue to become more powerful and swift in their calculations, the use of CFD will expand to new applications.

The technologies available today are leaps and bounds ahead of the resources available to those designing the MSBR in the 1960's. Computing power has made hydrodynamics and heat transfer incredibly more accurate in comparison to hand drawings and manual calculations done in the initial MSBR design (Bettis, et al., 1967). The computer programs developed at ORNL for optimization of the heat exchanger designs were rudimentary compared to the commercial software packages available today. Performing CFD analyses in FLUENT, with automated optimization available in ANSYS Workbench, is much more sophisticated than the analog computer models used in the 1960's for overall plant operation (Burke, 1972) and heat exchanger design (Bettis, Pickel, Crowley, Simon-Tov, Nelms, & Stoddart, 1971).

Compact Heat Exchangers in a MSR

Materials technologies have also come very far since the 1960's, which may allow for alloys more advanced than Hastelloy N to be available. Hastelloy N created today can also be guaranteed to be of greater quality due to the improvement of measuring and manufacturing techniques. More complex fabrication techniques may allow for corrosion resistant claddings to be added to inner structural members in order to capture the advantages of low-corrosion cladding materials with high-strength inner materials such as ceramics. And most importantly, compact heat exchangers have been developed to allow for more efficient, complete heat transfer in volumes much less than what was possible in the past. The primary heat exchanger is a perfect component that can be updated with modern materials, technologies, and analysis to improve an otherwise obsolete design.

Various studies have been performed in attempts to update the intermediate heat exchanger that is downstream of the primary heat exchanger, mostly because its performance is strongly tied to the ability to employ more efficient conversion cycles, such as the Brayton cycle using supercritical CO_2 (Harvego, 2006). Updating the unique set of conditions a primary heat exchanger is subjected to, which shares only a few similarities with those experienced by the intermediate heat exchanger, will be a valuable study for the future of MSRs. The liquid-to-liquid primary heat exchanger will come with its own advantages and challenges when compared to the liquid-to-gas intermediate heat exchanger.

This report will serve as a complete study of the design of a primary heat exchanger for a molten salt reactor. Using ORNL's operating conditions as a basis for which the updated heat exchanger will also function, the compact heat exchanger designs can be fairly compared and contrasted against the completed shell-and-tube design. A study in materials issues behind each design, fabrication techniques necessary for each design, and creating a commercially viable means for implementation of such a design will each be carried out. This thesis will serve to update the primary heat exchanger design of the MSBR with modern technologies that will allow for improved performance.

Heat Exchanger Design

Heat exchangers have existed for several decades, and various texts have been written to summarize standard practices in the design of such heat exchangers. No matter the application, material, and geometry used, the design of a heat exchanger should be completed in a similar manner. Classic textbooks exist on exactly this subject and are explained here.

A common approach for heat exchanger design has been developed in recent years to provide a flowchart for the long and iterative nature of heat exchanger design (Shah & Sekulic, 2003). Although modern software tools have eliminated the need to manually perform several separate tasks, the general order of the design process is still followed. In addition to laying out a concise

design process for designing any type of heat exchanger, there are also strict classifications for the different types of heat exchangers. The design process will be presented here but will be followed directly and displayed in more detail in the body of this report.

The schematic for the heat exchanger design process is displayed below in Figure 6 to emphasize the complexity and inter-related nature of the steps in the process. In this study, process specifications are brought from the ORNL study on the Primary Heat Exchanger (Bettis, et al., 1967). Thermal and hydraulic design is performed in FLUENT. Mechanical design can also be completed in ANSYS Workbench using the outputs from FLUENT found in previous steps. Several of the final steps shown in the diagram in Figure 6 (Heat transfer and hydrodynamic performance, mechanical/structural design, optimized design options, trade off, and system-based optimization) are amassed in the multiple functions of ANSYS Workbench, including the Goal-Driven Optimization module. The theory and process behind this portion of the heat exchanger design is provided later in this report.

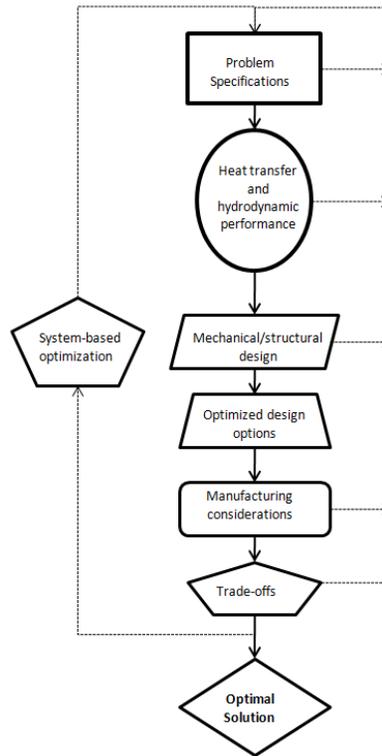


Figure 6: Heat exchanger design process (Kays & London, 1984).

In addition to the CFD design that encompasses the majority of Figure 6, manufacturing and economic considerations must be taken into account to complete a thorough heat exchanger design. For example, the possibility of casting the heat exchanger designed in this study is presented later in this report. Testing using ProCast, a Finite Element Analysis (FEA) program

that determines the feasibility of casting geometries with the proposed materials, is still needed to validate such a concept.

The process laid out in Figure 6 has been simplified by the ability of ANSYS Workbench to build, simulate, change, and optimize fluids and heat transfer problems. With manual input of the design specifications, ANSYS Workbench can provide accurate, optimized results on a significantly shortened time scale. Fundamental heat exchanger design texts were still vital in demonstrating a consistent process and identifying the numerous considerations that must be made.

II. Literature Review

Comprehensive resources are available for all aspects of the design of an advanced heat exchanger in the form of textbooks, educational papers, and formal project reports. The sources fell into three categories: compact heat exchanger basics, related ORNL studies, and compact heat exchangers in the nuclear industry.

Compact Heat Exchanger Basics

Several text sources cover specialized applications in compact heat exchangers that more thoroughly explain the nuances of the advanced designs and theory related to compact heat exchangers. In addition to outlining several key considerations and physical processes, these texts tend to look to the future by mentioning advanced concepts that may allow for improved performance. The following texts cover studies involved with the advanced compact heat exchanger designs being considered.

Fundamental texts have been written on the specialized scope of compact heat exchangers, which cover the key advancements in the industry and focus on the design issues that arise on the small scale in which they exist (Kays & London, 1984). Various issues arise in compact heat exchangers that are not present in shell-and-tube exchangers, which would provide especially challenging designs for use in the nuclear industry.

Of interest in this study are pure counter-flow compact channel heat exchangers, which can also be classified as “direct-transfer, indirect-contact” exchangers. This type of heat exchanger is characterized by two fluids transferring heat between each other without the fluids coming into direct contact. This type of heat exchanger is also known as a recuperator, describing the requirement to recover as much heat as possible from one fluid to the next. The current design also falls under “prime surface exchanger”, where no fins or extended surfaces are added to the

channels. Figure 7 shows the entire classification procedure used for heat exchangers with each category the heat exchanger in this study falls under shown in red.

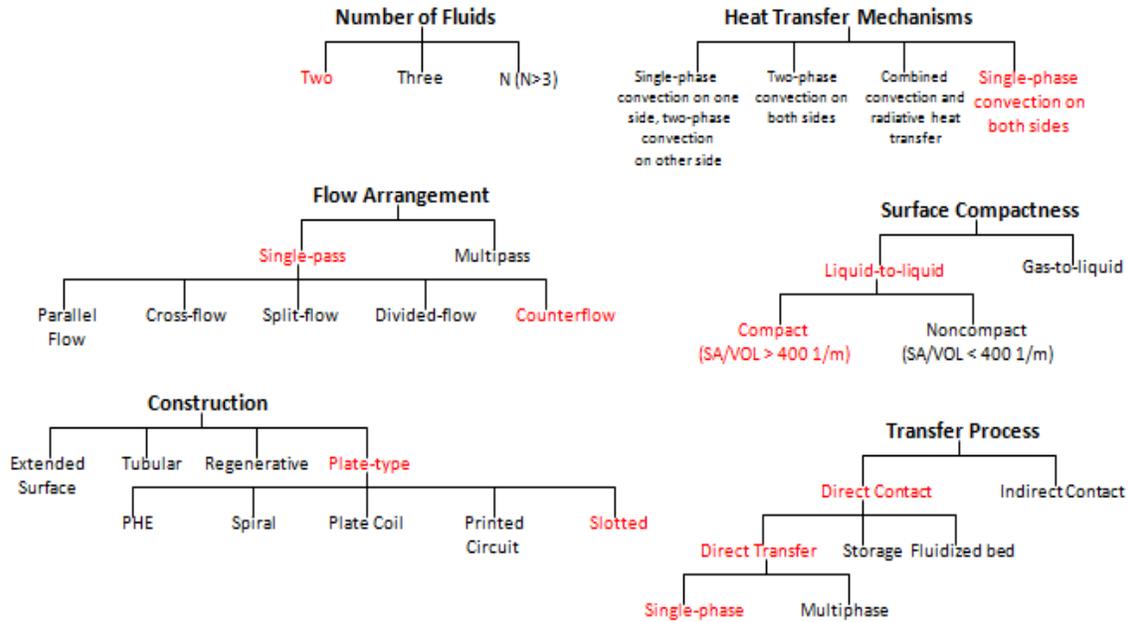


Figure 7: Classification procedure for heat exchangers, with the slotted minichannel shown in red (Kays & London, 1984).

As shown in Figure 7, a liquid-to-liquid exchanger must have a surface area density of $400 \text{ m}^2/\text{m}^3$ in order to be considered “compact”. If the heat exchanger designed in this study does not provide a significant upgrade in compactness and decrease in material usage, the complexities of using such a heat exchanger would not be warranted. Performing at a level sufficient to use the term “compact” heat exchanger will not be an issue in this study.

Correlations for channels of rectangular cross-section are provided for Nusselt number and Poiseuille number, which will be discussed later in this report as part of the process of validating the CFD model used in this study. Other similar correlations are available for “Plain Plate-Fin Surfaces”, which are plate-type heat exchangers that omit surface features and roughly resemble the rectangular cross-section channels that are the focus of research in this study. These studies show experience with rectangular channels in compact heat exchanger applications, but the specific boundary conditions prescribed in this study will not match the simple situations presented in the correlations (constant wall temperature and constant heat input on outer fluid surface). The correlations found for the physics in this study, which use an outer constant temperature boundary condition surrounding a finite array of channels, will be presented in the Results section of this report.

The calculations used in the Parameter Set of ANSYS Workbench shown in the Set Up section of this report are based on common heat exchanger parameters found in fundamental texts (Kays & London, 1984). The ϵ -NTU (Number of Thermal Units) and LMTD (Log-Mean Temperature Difference) methods are presented in detail, with the latter being exercised in this study.

The channel dimensions proposed in this research will fall within the “Minichannel” range due to restrictions on maintenance, which will invalidate microchannels (Kandlikar, Garimella, Li, Colin, & King, 2006). Channel sizes on the conventional range without surface features do not perform adequately well to justify their use in place of a shell-and-tube heat exchanger. The Minichannel range, especially at channel heights between 0.5mm and 2mm, is expected to be in the optimal range of balance between performance and maintenance. Table 2 displays the ranges of compact heat exchanger classifications by the size of the channel (Kandlikar, Garimella, Li, Colin, & King, 2006).

Table 2: Heat exchanger channel classification by size of hydraulic diameter (Kandlikar, Garimella, Li, Colin, & King, 2006).

| Channel Classifications | |
|--------------------------------|---|
| Conventional Channels | $D > 3 \text{ mm}$ |
| Minichannels | $3 \text{ mm} \geq D > 200 \text{ }\mu\text{m}$ |
| Microchannels | $200 \text{ }\mu\text{m} \geq D > 10 \text{ }\mu\text{m}$ |
| Transitional Microchannels | $10 \text{ }\mu\text{m} \geq D > 1 \text{ }\mu\text{m}$ |
| Transitional Nanochannels | $1 \text{ }\mu\text{m} \geq D > 0.1 \text{ }\mu\text{m}$ |
| Nanochannels | $D \leq 0.1 \text{ }\mu\text{m}$ |

An intriguing benefit that launched studies into a high-aspect-ratio, “slotted,” design is the high Nusselt number seen in such channels. Nusselt number, which correlates directly to the heat transfer coefficient for a given arrangement and generally compares the contributions of convective and conductive heat transfer, is an important value in considering the heat transfer performance of a design. With the effect of the Nusselt number on heat transfer performance considered to be greater in a final design than the Poiseuille number on pressure drop, an increase in the Nusselt number compared to a circular channel is attractive. Table 3 shows the effect of changing a rectangular channel’s aspect ratio on its heat transfer performance and pressure drop, and also serves to compare those values to those for circular channels (Kandlikar, Garimella, Li, Colin, & King, 2006).

Table 3: Fully-developed Nusselt correlations of varying channel cross-sections (Kandlikar, Garimella, Li, Colin, & King, 2006).

| Duct Shape | | | Nu_H | Nu_T | $Po = f Re$ |
|--------------|----------------------|----------|--------|--------|-------------|
| Circular | | | 4.36 | 3.66 | 16 |
| Flat Channel | | | 8.24 | 7.54 | 24 |
| Rectangular | Aspect ratio, b/a | 1 | 3.61 | 2.98 | 14.23 |
| | | 2 | 4.13 | 3.39 | 15.55 |
| | | 3 | 4.79 | 3.96 | 17.09 |
| | | 4 | 5.33 | 4.44 | 18.23 |
| | | 6 | 6.05 | 5.14 | 19.70 |
| | | 8 | 6.49 | 5.60 | 20.58 |
| | | ∞ | 8.24 | 7.54 | 24.00 |

The values found in Table 3 will be used later in this report to validate simulations made on FLUENT before moving to further testing for a rectangular cross-section heat exchanger. The values also provide motivation for changing channel geometry, as high aspect ratio rectangular channels appear to have significant heat transfer advantages, as shown by an elevated Nusselt number, with only minor deficiencies in hydrodynamics (as shown by a slightly elevated Poiseuille number).

Many design challenges and considerations that must be made in heat exchanger design are presented in Fundamentals of Heat Exchanger Design. Pressure and thermal stresses are the key detriments to structural stability of heat exchangers and must be considered equally with the loading stresses brought on by the weight of the exchanger (Shah & Sekulic, 2003). Transient thermal stress and the respective fatigue must also be considered due to expected changes, especially at start-up and shutdown. Flow-induced vibrations are a vital consideration that can be one source of erosion and fatigue in piping but may be of lesser importance in a monolithic structure. Large flow velocities can also cause erosion, corrosion, and fouling of channels and/or piping in the system. The main system components stressed in the text were the headers, manifolds and piping systems that will be necessary for the design. Many of the stresses and physical processes occurring within a heat exchanger are now simulated in the CFD program of choice. While many of the additional considerations are important and must be considered throughout the design process, their testing and validating are not required until much later in the design process.

In addition to the pressure drops that inherently occur across such devices, the geometry, sizing, routing, and arrangement of such flow distribution devices must be considered throughout the design of the exchanger. Manufacturing, equipment, and processing considerations must all be made to such a multi-faceted application. Trade-offs are present throughout the physics of any

heat exchanger design, so great detail must be taken to assure the proper decisions are made to meet all design criteria.

In order to make the design of a heat exchanger an infinitely more manageable task, several assumptions must be made for preliminary designs. According to calculations and simulations run after removing the assumptions, the importance of having such exactitudes can be weighed against any computational effort required to include them. Flow is assumed to be steady-state in order to hold true in order to avoid the complications of making transient calculations in preliminary studies. Heat losses are assumed to be negligible until an acceptable heat flux can be confirmed from experimental testing. It also must be assumed that no energy is stored or created in the fluids or walls in order to properly satisfy the energy equation. Single-phase flow must be assumed in order to further simplify any species calculations that may be required in multi-phase flow. Mal-distribution effects tied to irregularities in the inlet flow of the heat exchanger may be influential in applications, but will be neglected in CFD simulations.. Many assumptions have proven to commonly fail and must be considered in detailed designs. Modern, sophisticated computing takes care of nearly all of these potential issues.

The basics put forth in classic heat exchanger design texts have some aspects that have become obsolete with the rapid progression of commercial software packages. ANSYS Workbench, for example, is a modern CFD and multi-physics package that has optimization capabilities and automated iterative design capabilities that provide substantial improvements in accuracy and time of design problems. The details of interconnecting the classic heat exchanger design process with new software capabilities will be presented in a step-by-step approach later in this report.

Related ORNL Studies

Oak Ridge National Laboratory (ORNL) is the birthplace of the Molten Salt Reactor (MSR) concept, a technology that has gone untouched since its initial five- to ten-year period of rapid evolution in the 1960's. Researchers at ORNL conceptualized, designed, and tested several revolutionary components that would be necessary for the successful development of the MSR, and nearly all of that progress was left in concise technical documentation. Studies in chemistry, materials, manufacturing, and engineering were conducted to test the viability of the liquid-fueled design, as well as the multitude of design challenges that accompany such an advanced nuclear reactor design.

Oak Ridge gained its interest in liquid-fueled reactors when it initiated the Aircraft Reactor Experiment (ARE) in 1950. The ARE was a project that planned to use circulating molten salts to power a nuclear reactor in an aircraft (MacPherson, 1985). With the goal to lengthen flight

times indefinitely, the Air Force saw nuclear-powered aircraft as the flying equivalent to the nuclear submarine fleet the Navy was in the process of developing (LeBlanc, 2010). Interest in the aeronautical applications for molten salt reactors waned, but the technology showed enough promise during the ARE that research was launched into harnessing those abilities for civilian nuclear power from molten salts. ORNL Director, Alvin Weinberg, the foremost supporter of Thorium and molten salt technologies, proposed that the knowledge gained from the ARE be translated to an electrical-power-producing molten salt reactor.

As molten salt-related concepts continued developing, greater concentration was focused on the possibility of thermal breeding. The concept of a nuclear reactor being able to create its own fuel was intriguing for obvious reasons, so the 1960's saw a renewed interest in realizing the full potential of a thermal breeder reactor (MacPherson, 1985). The next step in ORNL's MSR reactor design culminated in the Molten Salt Reactor Experiment (MSRE), a step towards commercial relevance that remains perhaps the most influential researching tool ever created for the MSR technology.

ORNL launched the Molten Salt Reactor Experiment (MSRE), a single-fluid molten salt test reactor that produced just 8 MW of thermal power, in the early 1960's. With the creation of Hastelloy N, then called "INOR-8" during its developmental phase, corrosion issues with the molten salts were mostly mitigated. The MSRE was operated for five years to test materials, system components, and the stability of the reactor's operation.

While no issues were encountered during the entire operating life of the MSRE, corrosion found in the MSRE loop was enough to cast serious doubts for the long-term viability of molten salt reactors. Although the root of the corrosion issues was studied and believed to be found by the time of its cancellation, the Atomic Energy Commission (AEC) decided to cease major investments in MSRs (MacPherson, 1985). Liquid Metal Fast Breeder Reactors (LMFBRs) came in favor with the AEC and began their development, which proved to be the end of Molten Salt Reactor development.

Before research was halted on MSRs, ORNL conducted several studies on the design of the heat exchangers necessary in the MSR, including several detailed studies of the Primary Heat Exchanger (PHX). Many of the key design considerations brought forth by ORNL engineers during the design of their shell-and-tube PHX are addressed here and will play a vital role in the ability of any advanced PHX design to become feasible in power plant applications. In a similar process, albeit with much more advanced technology available than in the 1960's, this study will develop a PHX for the modern MSR. ORNL resources are available for the shell-and-tube heat exchanger design that was planned for the Molten Salt Breeder Reactor (MSBR), which will be used as the baseline design for this study.

ORNL conducted studies solely geared towards designing the heat loops and required systems for the MSBR. The PHX was a major component in several of such studies, and the evolution of

its design is well documented (Bettis, et al., 1967). This document, written and produced in 1967, did not have the modern advancements to analyze and design more complex designs than shell-and-tube heat exchangers. Hand-written calculations were augmented by small-scale experimental results to provide approximate values for design calculations. Thorough discussion of the considerations to be made for such a heat exchanger was provided, and will be a vast resource for similar concepts in the heat exchanger resulting from this study.

The work completed on the heat-exchange system of the MSBR was preceded by in-depth studies of reactor core characteristics and behavior that validated further work on molten salt reactor plant design. In order to understand the implementation of a Primary Heat Exchanger (PHX) in the broad scope of a molten salt reactor plant, a complete design of the MSBR concept must be explained.

The two-fluid MSBR design used four identical heat loops, each having one Primary Heat Exchanger, one Blanket Heat Exchanger, four boilers, two steam reheaters, and two reheat-steam pre-heater. Each loop carried 25% of the total plant load from the single 1000-MW(e) reactor. The modular nature of this design was conceptualized to allow for easier repair and removal of system components, such as the heat exchangers.

The reactor contained two separate regions: the graphite moderating region in the center of the core and the blanket region on the outside of the core. Fuel salt flows upward through channels in the graphite, where fission takes place and is controlled by the moderator. The blanket region reflects neutrons and produces some heat by fission itself.

The fuel salt leaves the bottom of the reactor at 1300°F, enters on the side primary heat exchanger near the top of its height, flows downward through the tubes of the PHX, and is cooled to 1000°F as it leaves from the side near the middle of the PHX's height. The cooled fuel salt is then returned to the reactor to be heated again.

The PHX is designed as a two-pass shell-and-tube heat exchanger with disk and doughnut baffles. The fuel salt flows downward through the tubes in the outer region of the heat exchanger and then flows upward through the tubes in the center region (Bettis, et al., 1967).

The blanket salt enters the reactor at 1150°F on the side of the reactor, flows in both directions around the moderator, is heated up to 1250°F, and leaves the reactor at nearly the same point it entered (Bettis, et al., 1967). Heat is removed from the blanket salt, and thus transferred to the secondary salt, by means of a blanket heat exchanger (BHX). The BHX and its necessary design is very similar to the PHX but has been ignored. After leaving the BHX and having its heat removed by the inert secondary salt, the blanket salt can once again re-enter the blanket region of the reactor at 1150°F.

Coolant salt enters the PHX at 850°F, flows countercurrent to the fuel salt through the PHX, and leaves at 1111°F. The pre-heated coolant salt then enters the bottom of the BHX, is heated by the

blanket salt, and leaves the top of the BHX at 1125°F. The coolant salt then is pumped to the secondary system, where it is used to perform two functions: to create steam in the boiler (87% of flow) and to reheat exhaust steam from high-pressure turbine (13% of flow). The coolant salt is then returned to its initial 850°F and re-enters the PHX to once again repeat the cycle. Figure 8 shows the entirety of one MSBR design created at Oak Ridge in the 1960's, which employs a two-fluid design attached to a steam cycle for Case A (an early PHX concept that was later improved and evolved into Case B).

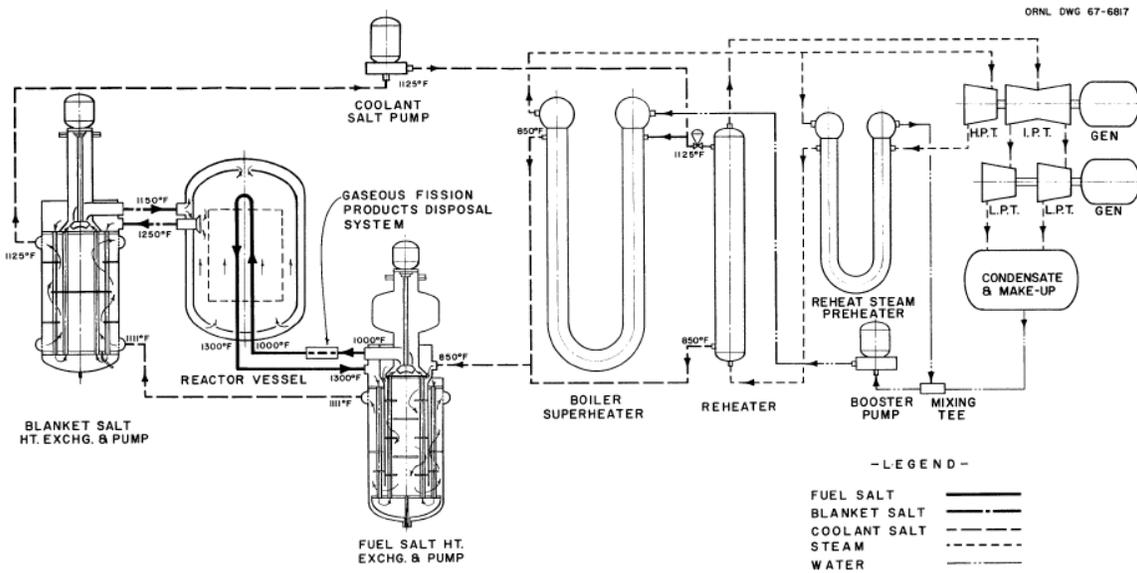


Fig. 1. Flow Diagram for the Case-A Heat-Exchange System.

Figure 8: MSR plant diagram showing boundary conditions, courtesy of Oak Ridge National Laboratory (Bettis, et al., 1967).

Two options for the PHX were considered, each of which have separate fuel salt and blanket salt sections, in addition to the secondary coolant salt. The design of each option was optimized and designed simultaneously, then contrasted at the end of ORNL's study.

The PHX designed in Case A planned for the fuel salt to leave the reactor and flow downward through the tubes in the outer section of the heat exchanger, flow upward through the tubes in the inner annulus, and return back to the reactor from the top of the PHX. The coolant salt flows in the opposite direction as the fuel salt on the shell-side of the heat exchanger. Coolant salt also enters at the top of the PHX, flows downward through the center section of the heat exchanger, flows upward through the outer section, and flows to the inlet of the blanket heat exchanger from the top of the PHX.

Figure 9 shows the original PHX design (Case A) planned for the MSR, with arrows showing the direction of fluid flow.

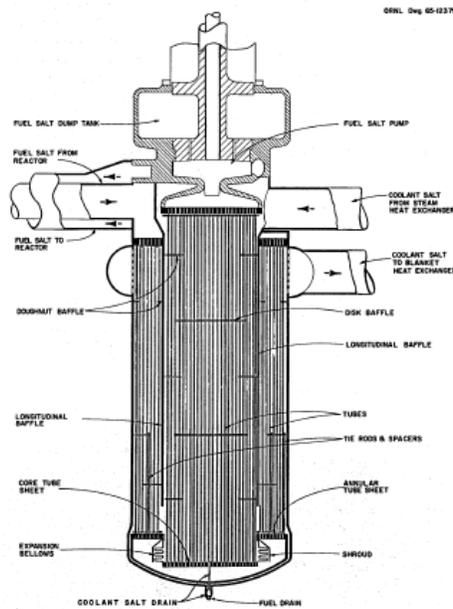


Figure 9: Shell-and-tube heat exchanger for Case A of MSBR design, courtesy of Oak Ridge National Laboratory (Bettis, et al., 1967).

Case A had issues arise when it required reheat-steam preheaters to raise the temperature of exhaust steam from the high-pressure turbine. Case B was developed to preclude the necessity for such a device. Case B also alleviated several issues that arose in the utilization of Case A's PHX, most importantly where operating pressures of the fuel salt were found to be higher than operating pressures of the coolant salt at similar points in the heat exchanger. This would cause fuel salt to leak into the coolant salt, in the event of a tube rupture, which would spread radioactive material to the secondary side of the plant. This issue was eliminated in Case B in favor of a more balanced pressure distribution (Bettis, et al., 1967). In making these changes, an advantage was seen in pressure distributions of the salts in the core. Lower pressures of the fuel salts were found throughout the core after the resulting design changes, especially in the areas where salts came in contact with the graphite moderator.

In the process of changing and rearranging the heat exchanger internals, the fluid inlets and outlets were altered slightly to adapt to the new layout. The difference is clear when comparing Figure 9 with the plant diagram of Case B shown in Appendix A.

Several changes were also required to the balance of the heat-exchange system due to the changes made in the change from Case A to Case B. All heat exchangers downstream of the PHX required slightly different temperatures and pressures to account for the changes in the system. The resulting system configuration is found in Appendix A.

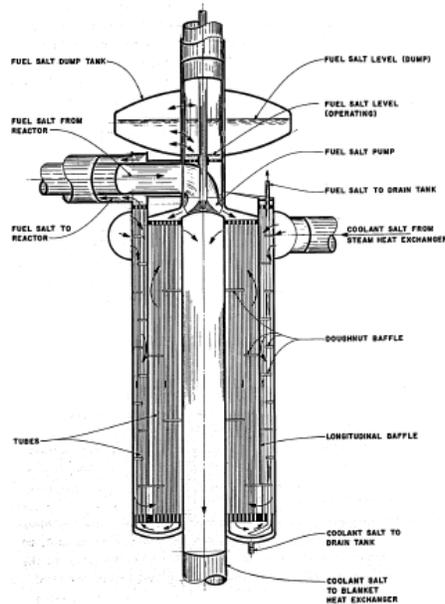


Figure 10: Shell-and-tube heat exchanger for Case B of MSBR design, courtesy of Oak Ridge National Laboratory (Bettis, et al., 1967).

Fluid flows were reversed in Case B, as opposed to how they were designed in Case A. In the Case B PHX, fuel salt enters in the center section at the top of the heat exchanger, flows downward through the tubes in the center section, flows upward through the tubes in the outer section, and leaves toward the reactor at the top of the heat exchanger. Coolant salt enters at the top of the heat exchanger in the outer region, flows downward through the baffles on the shell-side, flows upward through the baffles in the inner section, and then leaves toward the BHX at the top of the heat exchanger.

Table 4 provides the design parameters seen in each heat exchanger system and a description of the shell-and-tube design. These values will serve as the guidelines for which the new heat exchanger will be designed, in order to best contrast the compact heat exchanger design with a previously accepted design. The heat exchanger was made entirely of Hastelloy N, which at a size as immense as all four were planned to be, would be quite a capital expense.

Performance characteristics and geometric quantities were taken from this table to provide a concise means to which results from the compact heat exchanger can be compared. The results of such calculations will be presented later.

Table 4: Characteristics of ORNL's final shell-and-tube heat exchanger design (Bettis, et al., 1967).

| | | | |
|--|--|-----------------------------------|---|
| Type | Shell-and-tube two-pass vertical exchanger | Shell thickness (in) | 1 |
| Number required | 4 | Shell ID (in) | |
| | | Center section | 40.25 |
| | | Annular section | 66.7 |
| Rate of heat transfer, each | | Tube sheet material | Hastelloy N |
| | MW | 528.5 | |
| | BTU/hr | 1.8046×10^9 | |
| Shell-side conditions | | Tube sheet thickness (in) | |
| | Cold Fluid | Coolant salt | |
| | Entrance temperature (F) | 850 | Top outer annulus 1.5 |
| | Exit temperature (F) | 1111 | Top inner annulus 2.5 |
| | Entrance pressure (psi) | 198 | Floating head 3.5 |
| | Exit pressure (psi) | 164 | |
| | Pressure drop across exchanger (psi) | 34 | Number of tubes |
| | Mass flow rate (lb/hr) | $1.685E07$ | Inner annulus 4347 |
| | | | Outer annulus 3794 |
| Tube-side conditions | | Pitch of tubes (in) | |
| | Hot fluid | Fuel salt | |
| | Entrance temperature (F) | 1300 | Center section 0.600 |
| | | | Annular section 0.625 |
| | Exit temperature (F) | 1000 | Total heat transfer area per exchanger (ft ²) |
| | Entrance pressure (psi) | 146 | Center section 4875 |
| | Exit pressure (psi) | 50 | Annular section 4790 |
| | Pressure drop across exchanger (psi) | 96 | Total 9665 |
| | Mass flow rate (lb/hr) | $1.093E07$ | |
| Mass velocity (lb/hr*ft ²) | | Basis for area calculation | Tube OD |
| | Center section | 5.95×10^6 | |
| | Annular section | 5.175×10^6 | Disk and doughnut |
| Velocity (ft/sec) | | Type of baffle | |
| | Center section | 13 | |
| | Annular section | 11.3 | Number of baffles |
| Tube material | Hastelloy N | Center section | 4 |
| Tube OD (in) | 0.375 | Annular section | 10 |
| Tube thickness (in) | 0.035 | Overall heat transfer coefficient | |
| Tube length (tube sheet to tube sheet, ft) | | BTU/(hr*ft ²) | 1106 |
| | Inner annulus | 15.286 | Maximum stress intensity (psi) |
| | Outer annulus | 16.125 | Tube (Calculated) 6504 |
| Shell material | Hastelloy N | | Tube (Allowable) 17500 |
| | | | Shell (Calculated) 9945 |
| | | | Shell (Allowable) 56250 |
| | | | Maximum tube sheet stress (psi) |
| | | | Inner annulus 3500 |
| | | | Outer annulus 17000 |

A list of considerations and design factors was also made in the ORNL documentation (Bettis, et al., 1967) as a means for assuring that the heat exchanger design was of the quality necessary for its application. Although many of the considerations were made with a shell-and-tube heat exchanger in mind, the thoughts were valid and remain relevant for the compact heat exchanger of interest in this study.

The development of corrosion-resistant materials had already taken place at ORNL, and its use throughout the plant was considered. All metal surfaces that come in contact with the molten

fluorides in the MSBR system were required to be made of Hastelloy N, while other surfaces at elevated temperatures must have considerations made to address those points.

Just as maintenance is a large focus of the design of the compact heat exchanger in this study, the engineers at ORNL considered maintenance philosophy largely. The concept of using four separate trains, with separate heat exchangers for each loop, is a step in the process of becoming “modular” for ease of replacement and maintenance. In addition to making replacement easier with hot cells and smaller banks of heat exchanger modules, rather than making repairs in place, standard shell-and-tube exchanger maintenance philosophies are carried through to this design. Standard maintenance and cleaning procedures were carried from other shell-and-tube, but compact heat exchangers will require that new, innovative processes be developed.

All calculations by ORNL were performed manually, without the assistance of computers to provide three-dimensional modeling capabilities until much later. More complex, thorough models will be able to be performed now with the current technologies available using Finite Element Analysis (FEA). Automated, accurate optimization will make for heat exchangers with even greater performance.

Compact Heat Exchanger Developments

In addition to the reports provided under large research programs that have been under way in recent years, several educational papers have been written covering the analytical aspects of compact heat exchanger design. Whereas project reports have a focus on creating heat exchangers for a test apparatus or a full-scale application, the more specialized technical papers cover different, albeit similarly important, aspects of the heat exchangers. The study and development of heat exchanger correlations and advanced methods of computation developed in these papers and texts has been vital in the advancement of compact heat exchangers. Several of these sources will be presented here.

A comprehensive text about one type of compact heat exchanger, the plate-type heat exchanger (PHE), has been developed to introduce the wealth of options available for construction of this type of heat exchanger (Wang, Sunden, & Manglik, 2007). Plate-type heat exchangers are named for the manner in which they are produced, where plates are stacked directly on top each other to create the cross-section of the heat exchanger channels. Bonding techniques are required for plates to adhere to one another, which is often achieved by brazing or welding. This requirement brings forth a weakness in the mechanical integrity of the system and may be able to be eliminated with casting techniques. Although the manner in which the channels are produced may differ in the results presented later in this report, the findings for channel sizes and shapes remain relevant for any compact heat exchanger.

There are many different types of fabrication methods that are categorized under plate-type heat exchangers: brazed plate, fully welded, wide-gap, double-wall, diabon graphite, and minex plate heat exchangers. Each type was specifically designed for different temperature and pressure restrictions, none of which are desired in a molten salt reactor.

Many plate-type heat exchangers use gaskets to connect plates, which would render them useless in high-temperature nuclear applications. The text mentions that plate heat exchangers can be 20-30% of the size of standard shell-and-tube heat exchangers, which would obviously present a large improvement in capital costs. Table 5 displays a comparison of abilities of a Gasketed PHE, a plate-type heat exchanger with plates joined by gaskets at their interface, compared to a shell-and-tube heat exchanger.

Table 5: Comparison of Gasketed Plate Heat Exchanger with shell-and-tube heat exchanger (Wang, Sunden, & Manglik, 2007).

| | Gasketed PHE | Shell-and-tube HX |
|---------------------------|------------------------------------|---------------------------------|
| Direct fluid contact | Impossible | Possible |
| Approach ΔT | $\sim 1^\circ\text{C}$ | $\sim 5^\circ\text{C}$ |
| Multiple duty | Possible | Impossible |
| Piping connections | From one direction | From several directions |
| Heat transfer ratio | $\sim 3-5$ | 1 |
| Operating weight ratio | 1 | $\sim 3-10$ |
| Hold-up volume | Low | High |
| Space ratio | 1 | $\sim 2-5$ |
| Welds | None | Welded |
| Sensitivity to vibrations | Not sensitive | Sensitive |
| Gaskets | On every plate | On each bonnet |
| Leak detection | Easy to detect | Difficult to detect |
| Access for inspection | On each side of plate | Limited between tubes |
| Disassembly time | ~ 15 minutes | $\sim 60-90$ minutes |
| Repair | Easy to replace plates and gaskets | Tube plugging hurts performance |
| Thermal size modification | Easy, add or remove plates | Difficult |
| Fouling ratio | $\sim 0.1-0.25$ | 1 |

Welds and gaskets have been replaced recently by more robust ways of connecting plates, such as diffusion bonding, but the idea of using very small channels manufactured by stacking similarly shaped plates remains valid. More sophisticated bonding techniques will be necessary

to provide ample strength at such high temperatures, but manufacturing the modules as plates would simplify manufacturing, allowing for commercial production. Although the fabrication method may differ from those presented in the text, some intriguing plate-type heat exchanger designs were presented.

The wide-gap plate heat exchanger was presented in this reference and is an intriguing design that has similarities with the heat exchanger designed in this study. Wide-gap PHEs are often used in applications where clogging may become an issue, such as some juices in sugar mills and paper mills. The wide-gap PHEs presented in the reference, which are designed by Alfa Laval, have complex surface corrugations and wavy patterns to enhance heat transfer. Such surface features will be removed for the purposes of this study, but the existence of such an intermediate design confirms that a concept that falls between very compact micro-channel heat exchangers and shell-and-tube heat exchangers is valid.

Modern achievements in the development of compact heat exchangers have been brought together in text form to review the possibilities contained by the devices (Sunden & Shah, 2007). This reference introduced several different types of compact heat exchangers, with a great deal of detail in new fabrication techniques. This book acknowledges molten salts as a possible working fluid in compact heat exchangers and introduces some concepts that could be compatible.

Several issues still exist with ceramic heat exchangers, with the most pressing needs being the following: ceramic tubes are expensive, long tubes with constant shape and properties are difficult to fabricate, tube-to-tube sheet joints are not yet leak-tight with current fabricating techniques, durability in corrosive environment is yet to be confirmed, the coefficient of thermal expansion may change with time, and mechanical properties of ceramics are not very well known at this time. It seems that many of the issues with ceramic heat exchangers involve manufacturing and materials, rather than heat transfer and/or fluid flow. These concerns will delay the time frame in which ceramics are viable in heat exchanger designs, so the use of such materials will not be studied in this report.

Significant effort and research have gone into purely theoretical studies that intend to optimize surface features in compact heat exchangers (Kanaris, Mouza, & Paras, 2004). The report covers modern enhancements in compact heat exchanger channel geometry technology that can now be modeled on computers in CFD programs. Significant research has been completed to optimize the addition of corrugations, each of its own channel aspect ratio, corrugation aspect ratio, and angle of attack, in order to best enhance heat transfer and minimize pressure drops. The focus of this study will not include such features, but future iterations can employ them if maintenance is still deemed reasonable.

The studies presented in this source have shown that drastic improvements in heat transfer coefficient and heat transfer area can be achieved by including surface corrugations on the inside

of the channels, while only modest increases in pressure drops are seen. Several reports have been written on different processes by which optimization can be achieved for different shapes and sizes of corrugations, but this will not be the focus of this report. If a simple, un-optimized compact heat exchanger can perform at a level near that of well-known shell-and-tube heat exchangers, it should be clear that such a technology holds significant promise in the future. Although the heat exchanger design in this study will not take advantage of the complexities that can make compact heat exchangers incredibly efficient, the possibility exists to add such features to increase performance.

Nuclear-related Compact Heat Exchanger Research

Interest in high-temperature applications in the nuclear power industry have led to research involving the use of molten salts as secondary coolants. Interest in pairing advanced nuclear reactors with the Brayton power cycle has led to research on intermediate heat exchangers (IHX) that transfer heat from a molten salt coolant to helium or super-critical CO_2 . Although the results of these studies are not directly applicable to a study of the PHX, the processes and considerations made for the design of each heat exchanger are similar. The following reports were investigated for applicable information.

A cooperative research effort between MIT and Heatric, a compact heat exchanger design company, was based on their Printed Circuit Heat Exchanger (PCHE) design and produced an early set of results for high-temperature compact heat exchangers (Gezelius, 2004). The specific design in this paper was for an intermediate helium-to-helium heat exchanger, but several key considerations and concepts will hold true for all nuclear-based heat exchanger design.

A key part of this paper was the header design created by Heatric that included built-in headers. The multi-ported printed circuit heat exchanger (MP PCHE) would have less mal-distribution effects, better fabrication techniques, easier assembly, and less associated piping. The ability to construct the header in a similar “plate-type” manner to the rest of the heat exchanger construction is an advantage in construction and fabrication, and its effects of minimizing pressure drop are yet another reason to pursue a similar concept.

Most of the work referenced was properly modeling the wavy channels used by Heatric to optimize heat transfer while minimizing pressure drop. Heatric has not made their proprietary design public and there was not a readily available heat exchanger for testing, so the author had to work to create a reasonably similar computer model that could be used to perform optimization and sensitivity analyses. Due to not knowing exact information for many of the design parameters used in his study, the author does “optimistic”, “mean”, and “pessimistic” evaluations for each case. In the end, the PCHE was found to be superior to all other designs, with the only modest disadvantage being in slightly increased pumping power (due to a slightly higher pressure drop).

For the PCHE, the maximum size for a module was listed at 600mm in height, 600mm in width, and over 1.5m in length. These constraints may have changed since 2004 and will differ for other manufacturing methods and locations, but it gives a rough estimate of how large heat exchangers can be made for that fabrication method. The diffusion bonding process used to join plates provides a monolithic structure of the same strength as the parent material, so creep will not be an issue. While avoiding such a process with a monolithic casted structure would be optimal for the design presented in this thesis, diffusion bonding remains a viability possibility for joining sections if complete casting is not possible. Heatric aims to make the incurred pressure drop ~1% of the operating pressure of the system, a viable value to use for all designs with different pressure values.

Wavy channels, especially of the extremely small scale that these were made for the PCHE, are not similar to the design points used in this study. The performance of the PCHE may be better than the slotted design explained in this thesis, but the issue of maintenance could be insurmountable for designs similar to the PCHE. The slotted minichannel heat exchanger design introduced later in this thesis will take into account such details.

Idaho National Laboratory (INL) completed a significant amount of work on several high-temperature heat exchangers as far back as 2006, when studies were conducted on three heat exchangers (the first heat exchanger downstream of the reactor, the heat exchanger that connects the balance of plant to the Hydrogen production loop, and a secondary heat exchanger that may or may not be necessary), none of which apply to research with a MSR's PHX (Harvego, 2006). The design is used for a very high-temperature reactor, so explanations for increasing the overall reactor temperatures are included. The paper describes the outlet temperature as the most influential parameter for determining overall heat transfer efficiency in the design. Overall plant efficiencies of up to 51.4% have been postured for plant temperatures up to 950°C, a stark improvement over current LWR efficiencies. The author explained the operating limits of a PCHE to be compactness of up to 2500m²/m³, operating temperature up to 900°C, and operating pressure of up to 50 MPa.

A heat exchanger effectiveness of 95% was required for each potential design, and then the design parameters were altered to optimize other heat exchanger characteristics. The inlet and outlet parameters were seen as more flexible than allowing for poor performance, which is an interesting outlook to take on any heat exchanger design. The operating conditions prescribed by ORNL are more rigid than those explained in the INL design, so such an outlook will not be possible for the work presented in this thesis. A temperature difference of 360°C in the core was used for all designs, while higher temperature drops may be possible in future designs. The author also considered raising the operating pressure to around 3 MPa in a PCHE, in order to decrease the required pumping power. This is still a relatively low pressure compared to current LWR conditions, and is well within the 50 MPa limit for PCHEs.

More recent work has been performed at Idaho National Laboratory that also covers the design of an intermediate Helium-Helium heat exchanger for the High Temperature Gas-Cooled Reactor (Mizia, 2008). Although the precise design and requirements differ for this specific heat exchanger and plant design, some valuable information was gathered from this report.

The study was restricted mostly to a multitude of possible metal alloys, but also considered foam, capillary, and ceramic heat exchanger designs. There was an excellent introduction with different heat exchanger possibilities listed, and the report even goes into detail about current and past reactors of similar designs. Concepts involving multi-temperature-stage construction and modular heat exchanger banks were presented and considered for the PHX. Multiple temperature stages would allow for different alloys to provide their respective advantages as required throughout the heat exchanger. Alloy 617 is proposed for the high-temperature section due to its excellent performance at high temperatures and increased resistance to corrosion, while Alloy 800H can be used for the low-temperature section of the heat exchanger that will not be under such harsh conditions. The possibility of using modular sections to construct the heat exchanger was briefly mentioned but only as an interesting concept for the future. Although the concept of only using rare alloys for the high-temperature section of the heat exchanger where its use is required, the design challenges in manufacturing and differential thermal expansion may be prohibitively serious. The concept may also not be practical when testing requires Loss of Coolant Accident (LOCA) simulation, as would be required during commercial plant designs. LOCA simulation would bring all temperature channels to elevated temperatures that would render the low-temperature material insufficient. These practices must be accounted for in all heat exchanger designs and seem to prohibit multiple-stage construction from being a valid design.

The above study had a primary focus on corrosion and its effect on the system and was found to be potentially life-limiting at high temperatures. Hastelloy XR, which is used in Japanese High-Temperature Test Reactor (HTTR), was mentioned as a possible corrosion-resistant material that may be an option for cladding over a stronger base material.

Eight types of heat exchangers were explained and evaluated on parameters such as compactness, maturity, stress behavior, and sensitivity to corrosion. Tubular and plated-stamped Intermediate Heat Exchanger (IHX) designs were chosen for further evaluation and testing.

Minor details from experimental results were introduced, with one pertinent observation regarding the operating shape of the channel being quite interesting. Originally square in cross-section, the channels were found to be slightly trapezoidal during operation due to the high temperature change. This is a somewhat drastic change that could be designed for in the future and also represents one of several design changes that will be incurred when moving from CFD simulations to experimental setups. The effects seen from extremely high temperatures changes cannot be discounted in experimental testing.

The progress made among the consortium designing the Intermediate Heat Exchanger (IHX) for the Next Generation Nuclear Plant (NGNP) has been well-documented and provides a wealth of knowledge on recent advancement in compact heat exchangers (Hechanova, 2007). The project focuses on very high-temperature reactors that would function to produce Hydrogen as a result of its high-temperature fluid streams, with electrical production also being a vital byproduct of the system. Among the groups working together on the project are the University of California – Berkeley (UCB), the University of Nevada Las Vegas (UNLV), General Atomics (GA), and Ceramatec Inc. This particular Progress Report was relatively far along in the development process, so extended work has been contributed by each party.

The UNLV group has identified three possible types of heat exchangers (Ceramatec compact heat exchanger, bayonet-type, and shell-and-tube) for comparison. Heat transfer and stress analyses were performed numerically for the heat exchangers, while the use of different metal alloys was the focus of the study.

UCB studied the use of ceramics (CSiC/C composites) as a means to allow for temperatures up to 1000°C in compact heat exchangers. The UCB group used the Compact Heat Exchanger Explicit Thermal And Hydraulics (CHEETAH) code to perform the heat transfer and stress analyses for compact heat exchangers made by Heatric. A large focus of their studies was making composite materials and coatings that could withstand the thermal transients and stresses, while remaining economically competitive with commonly used metal alloys. The ease and ability of construction and fabrication were also key factors studied by the UCB group.

General Atomics performed crack and corrosion testing, but its focus was on the sulfur-iodine process for hydrogen generation proposed in their high-temperature reactor design.

This group is responsible for the vast majority of recent nuclear-related heat exchanger updates that have occurred since ORNL designed the shell-and-tube heat exchanger for the MSBR. The NGNP reactors for which these heat exchangers were designed operate at very high temperatures in highly corrosive environments, much like the MSRs studies decades ago. Significant studies into the viability of compact heat exchangers have been made recently in an effort to update components for future plants, but the PHX has not seen any updated efforts at all. Since it is only relevant when Molten Salt Reactors are used, which make up only one option of the several candidates for the NGNP, the research efforts for the PHX have remained very limited.

The United States is not alone in its interest to develop compact heat exchangers in new industries, as seen in work from the Netherlands from 2008, which covers the used of compact heat exchangers for the Helium-Helium secondary heat exchanger in the High Temperature Reactor program (Pra, Tochon, Mauget, Fokkens, & Willemsen, 2007).

The exact results from this study are rather limited for the focus of research in this report, but a thorough study of thermal transients was completed. The most intense transient was deemed to be losing power from off-site, where the temperature drop on the low-pressure inlet is 335°C in 5

seconds. Transients and resulting thermal shocks are a key consideration that must be accounted for, and the most intense are tested to confirm that worst-case scenarios are acceptable.

Advantages and disadvantages are seen when using wavy surfaces inside the pipes, where the maximum advantage is seen in turbulence transition and the minimum advantage seen is highly-laminar flow. Correlations are given for the Nusselt number and friction factor for wavy channels. The focus of this thesis will not be on the optimization of the wavy channels, as several researchers have already completed such studies.

The heat exchanger in this study from the Netherlands has been designed to be tested on the CLAIRE loop, a high-temperature test loop that uses air as the working fluid due to interest in liquid-to-gas heat exchangers. An in-depth explanation of the test apparatus is given to explain the process and location for taking measurements. Such information would become very useful if a test apparatus is built to test the compact primary heat exchanger in molten-salt-related applications.

The CFD analysis used by the author was explained in detail, as one set of hot and cold passages was modeled to begin the study. With advanced computer modeling, thermal transients that are similar to conditions that would be seen during operation can be tested to see how the system responds. From the data received, the magnitude and location of maximum stresses can be calculated and designed for; however, the decision to model only a few passages may be incomplete in explaining the physical processes occurring throughout the heat exchanger. When hundreds of tubes are necessary in multiple directions, some intricacies in the flow may not be captured by only modeling two tubes.

The largest thermal stresses in compact heat exchangers are found at the inlets and outlets where rapid, large fluctuations in temperature were seen during a cold shock. Accommodating this area of increased stress with thicker channel barriers at the ends may be a useful design option. The transition between channel geometry and outlet materials is an important place to consider thermal stresses, as well as hydrodynamic properties such as pressure drop.

The results were found to be encouraging from the study. The process of scaling the design up to a more representative array size will require significantly increased computational load. Flow mal-distribution and conduction wall effects may be more influential on larger scales and must be considered. Expanding the computer model may help to alleviate these types of worries.

Recent studies have been completed with a focus on the ability of computer models to accurately model small geometries and large temperatures on rather large scales (Figley, 2009). The paper contains the design for a molten salt-Helium heat exchanger for a test loop to be constructed at Ohio State University (OSU). The heat exchanger is designed as a Printed Circuit Heat Exchanger, which describes its unique manufacturing process that is borrowed from the circuit board industry. The heat exchanger is to be used on a Very High Temperature Reactor that is

graphite moderated and helium cooled using a once-through uranium cycle to produce 600 MW of heat. FLUENT was used to model and simulate the designs.

Turbulence added a great deal of complexity to the calculations and correlations used in the study at Ohio State. The LMTD method was used to convey the overall performance of the heat exchanger. The mass flow rate was manipulated within a reasonable range to find the resulting pressure drop, overall heat transfer coefficient, average convective heat transfer coefficients, NTU, and heat exchanger effectiveness. Detailed graphs and charts show the results, with more detail going into a perceived region of optimization from 35-45 kg/s.

The Ohio State loop will operate at a maximum temperature between 900 and 950 °C. The Helium in the loop will operate at up to 3 MPa, 10-80 kg/hr, and inlet temperatures of 900 and 540 °C for the hot and cold fluids, respectively. Heat transfer coefficients between 563 and 1697 W/m²K are seen in the results through the entire range of mass flow rates. Effectiveness of 85% was reached for certain designs in the study. The maximum pressure drop was only 1.5% of the operating pressure, which was deemed acceptable for the purposes of the study. This study performed optimization based on geometric changes and mass flow rate variation, which will be very similar to what is done later in this thesis.

The paper explains that compact heat exchangers usually operate from laminar to low-Reynolds number turbulent region, in order to minimize the pressure drop that will induce high required pumping powers. As explained previously, PCHEs have very small flow channels that are etched into flat metal plates using a photochemical machining process. This process was borrowed from the electronics industry, which uses it to make circuit boards. The geometries this method of fabrication seems somewhat limited, as a semi-circular shape was used (changing some correlations and critical values). The diameter was to be kept between 0.5 and 2 millimeters.

The study began with a simplified bank of 10 hot plates and 10 cold plates, with each plate only containing one flow channel. A more representative full model was later developed that had 12 channels in each plate, while maintaining the same number of overall plates (20).

The testing process and technology for computing temperature distributions and the physical processes was excellent, but it failed to connect the work with the manufacturing and construction issues that are just as important in a final design.

FLUENT 6.3.33, which uses Gambit to create the geometry and mesh, was successfully used in the above study. Due to the massive amount of points necessary in the mesh, the author had to split the heat exchanger up into several pieces, each of which was computed separately and joined by boundary conditions. The massive computational resources available to the author would render results more accurate and detailed than those presently available to complete the research presented later in this thesis, so more care must be taken to efficiently use mesh nodes and resources.

Several advancements and focuses taken on in the study at OSU will not be directly related to those used in the research explained later in this thesis. The primary heat exchanger (PHX) will require different conditions than the intermediate heat exchanger (IHX) required in research at Ohio State. The focus on numerical studies and optimization presented in the IHX study were not the focus of the industrial, application-based study for the PHX in this thesis (or other applications requiring more rigorous maintenance and inspection programs). The boundary conditions for a PHX were previously prescribed by ORNL during studies of the PHX and were rigid due to the requirement to change other aspects of the design with large temperature and pressure discrepancies. The IHX had much more freedom in allowing boundary conditions to change in order to attain optimal performance.

Several sources provided keen insight into common procedures and issues that may arise during the design process. Although the application of the heat exchanger design in this study is different than those previously introduced, the resources available played a large part in shaping the decisions made for the slotted heat exchanger designed in this study.

Through examination of previous studies in heat exchanger basics, ORNL's progress, and compact heat exchangers, a modular "slotted" heat exchanger design, which uses thin rectangular channels on the scale of 0.4 mm to 5 mm in height, will be studied in detail to arrive at an optimal arrangement. The considerations made in previous studies, in conjunction with the set of conditions seen with a PHX in a MSR, will shape the final design of the heat exchanger created in this study.

III. Design and Setup

The body of this thesis will describe the ideal heat exchanger design and the process by which analyses were performed on the narrowed scope of prospective heat exchanger designs. An explanation of FLUENT and its necessary steps, as well as the results from those simulations, will be the focus of this section.

Design Considerations

The PHX to be designed in this study will attempt to find a middle-ground between the ultra-compact, ultra-efficient micro-scale compact heat exchanger and the well-known, easily maintained shell-and-tube heat exchanger. Combining the improved performance of compact heat exchangers, along with the robustness and ease of maintenance available with shell-and-tube designs, would make for an intriguing heat exchanger design.

The relative importance of the considerations laid out in this section will vary based on the Molten Salt Reactor application in the nuclear industry. The burden of using advanced forms of power generation comes with an added responsibility for safety that must be maintained in any nuclear-related concept. Considerations for maintenance, corrosion, plant layout, manufacturing, and materials will now be explained.

Maintenance and Inspection

Maintenance is an extremely important consideration in nuclear applications, and attention must be paid to ensure that reasonable and accurate methods can be used to monitor and predict damage to heat exchanger channels.

New heat exchangers should be able to provide similar means of maintenance by Non-Destructive Testing (NDT) similar to LWR steam generators and heat exchangers, which are currently maintained mostly by eddy current inspections. NDT techniques have evolved greatly in recent years as a means to eliminate costly replacement and removal of vital system components. The correct implementation of NDT can ensure quality, reliability, and safety for the system at hand (Rao, 2007). The ability of eddy current inspections, liquid penetrant testing, and other NDT techniques to predict future failures provide another benefit that makes it a necessity in plant maintenance.

Creating a new compact heat exchanger design will require research and development of new maintenance and inspection techniques that may not carry over from shell-and-tube designs. Maintaining some aspects of the well-known eddy current inspection process would be a distinct advantage for a new heat exchanger design, so focus was made to ensure that new designs could have simple maintenance procedures developed. With a greater knowledge of the physics involved in eddy current testing, a better understanding can be developed for the viability of using eddy current inspection with different heat exchanger designs.

Eddy current inspections function by inducing magnetic eddy currents in conducting solids and examining the resulting changes in the solid material. Changes are observed on the surface of the conducting material when defects cause the magnetic flux to change. As the flux changes, small circular patterns of currents (called “eddy currents”) are seen perpendicular to the area where flux is changed. This method can serve to perform several tasks, but finding cracks and defects is among its most important uses for NDT of heat exchangers (Rao, 2007). Figure 11 shows the process used in eddy current inspection, in which actuators for the pancake assembly were added to the excitation stage (Rao, 2007).

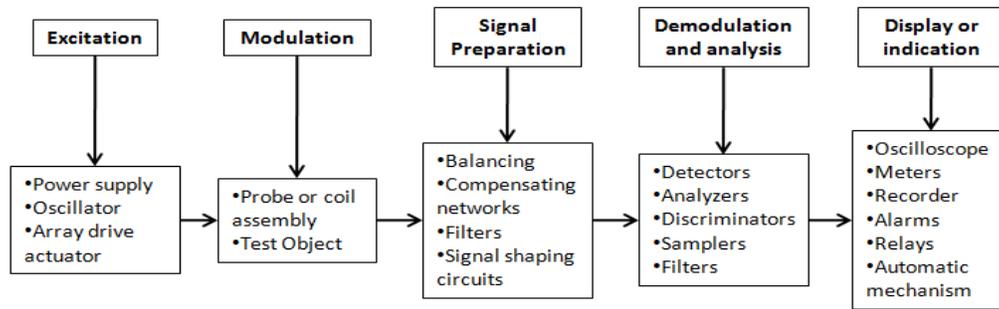


Figure 11: Eddy current inspection process schematic (Rao, 2007).

In a once-through straight-channeled heat exchanger design, gaskets could easily be removed from the inlet and outlet of the header on each side of the heat exchanger length. Arrays of eddy current probes could be lowered through the plenum and into the heat exchanger channels, and then lowered throughout the length of the heat exchanger module. Such a process would be easier to complete and repeat than that of a shell-and-tube heat exchanger because of the relative simplicity of the headers and channels.

Limitations exist with eddy current testing to completely examine heat exchanger conditions, mostly linked to the inability to perform deep penetrations into some conductors. The thin walls of the slotted PHX design in this study should mostly alleviate that issue, but eddy current testing is usually used in conjunction with other techniques that are able to provide more in-depth local results after eddy current testing finds potential issues. Channel surfaces have been kept plain in initial studies to further facilitate the use of eddy current inspection along the length of the heat exchanger. Wavy channels, which would still allow for an eddy current probe to be in direct contact with the HX channel along its entire length, are a future option for inducing turbulence and improving heat transfer. With constant channel gaps maintained with a wavy design, eddy current inspection is not expected to be adversely affected. As testing is developed on such a design, new devices could be developed to perform localized testing, as required.

Medical applications have shown that circular eddy current probes can function at diameters approaching roughly 1.8 mm (Libby) by using high-frequency signals in testing. The dimensions proposed in this study will fall below that current limit of circular eddy current coils, so further studies will need to be made to validate the use of eddy current probes on the required scale to make the heat exchanger performance competitive. Although inspection techniques are not yet readily available on such an exchanger design, it is believed that even higher frequencies can be used to reach channel dimensions as low as 0.5 mm.

Other advancements that will be required as a result of the design in this study will be high-aspect-ratio pancake-style eddy current probes on the relatively small scale explained in this design. Issues will arise with the sensitivity of the pancake probe at abnormal locations, especially at corners in a purely rectangular design, but it is expected that provisions can be designed for more in-depth NDT studies.

The research and development required to create and validate such equipment is expected to be minimal in comparison to the advantages seen by the performance of the heat exchanger. Just as time will be required to test and validate the heat exchanger design on a commercial basis, new and improved maintenance techniques will be required for such designs. Improvement upon reliability and accuracy of testing could be another advantage of the new techniques required for a compact heat exchanger design.

Corrosion

Molten fluoride salts, such as FLiBe, are not highly corrosive materials when paired with corrosion-resistant alloys like Hastelloy N. Studies have shown nearly negligible corrosion rates (less than 0.01 mm per year) from FLiBe with Hastelloy N at 600 °C for extended periods of time. Fuel salts, which are known to be more corrosive, due to the addition of oxidants such as UF_4 , still only have corrosion rates of approximately 0.02 mm per year (Sohal, Sabharwall, Sharpe, & Ebner, 2010).

Care must be taken to design a heat exchanger that is not susceptible to such corrosion rates or corrosion buildup in heat exchanger channels. If any type of blockage occurs, by means of corrosion products plating out or loose impediments becoming stuck in flow channels, they can severely damage the performance of micro-scale heat exchangers. Considerations must be made to ensure that channels are robust enough to deal with regular occurrences such as these. Channel thicknesses must remain thick enough to remain intact during long-term erosion that may occur during the life of the exchanger.

Although corrosion results were found to be negligible for clean FLiBe with Hastelloy N, the fissile-material-containing fuel salt was found to cause noticeably more corrosion. Rates of up to 0.01 mm per year were witnessed in studies using Thorium-containing fuel salts, which could provide a means for accident over long periods of use. Thus, the heat exchanger modules may need to be replaced periodically over the life of the reactor plant, and the cost for doing so as necessary will present an advantage for the modular slotted heat exchanger design.

Micro-sized filters at inlets and outlets must be provided, and channel sizes and wall thicknesses should be chosen to appropriately handle any corrosion issues and/or loose impediments. A heat exchanger wall rupture would contaminate the secondary side of the entire plant, and dimensions must be designed conservatively to assure that such an accident could never occur.

Plant Layout

ORNL experimented with creating four separate modules to split the heat load and require more cost-effective replacement of major components in the MSBR. The benefits of creating less capital investment when one module must be replaced can be offset by additions to piping,

valves, and other necessary components. Figure 12 shows an overview of the reactor modules in the plant layout of the MSBR.

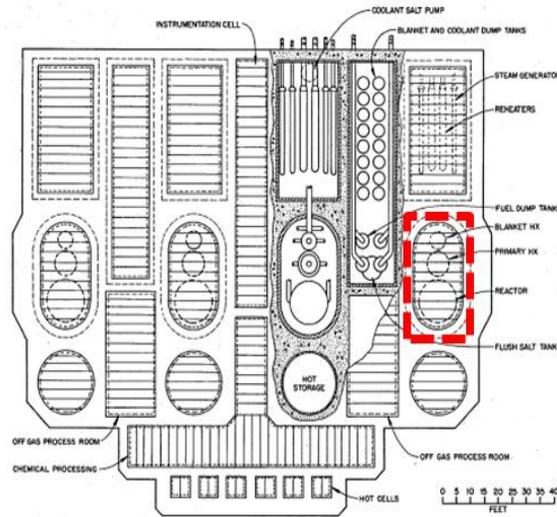


Figure 12: Top view of single-fluid MSR plant layout, courtesy of Oak Ridge National Laboratory (Robertson R. C., 1971).

The red box shown on the diagram in Figure 12 shows one reactor module, with the reactor shown next to the PHX and BHX. The structure is known to be 24 ft wide by 40 ft long by 63 ft deep (Rosenthal, Briggs, & Kasten, 1967).

As will be discussed later in this report, the area of such a structure using intelligently placed compact heat exchanger modules along reactor walls could be substantially reduced. Instead of a long rectangle with rounded edges, a square cell could be created to closely wrap around the reactor.

An elevation view of the reactor structure for the MSBR is shown in Figure 13, with the red box highlighting the reactor cell (Rosenthal, Briggs, & Kasten, 1967).

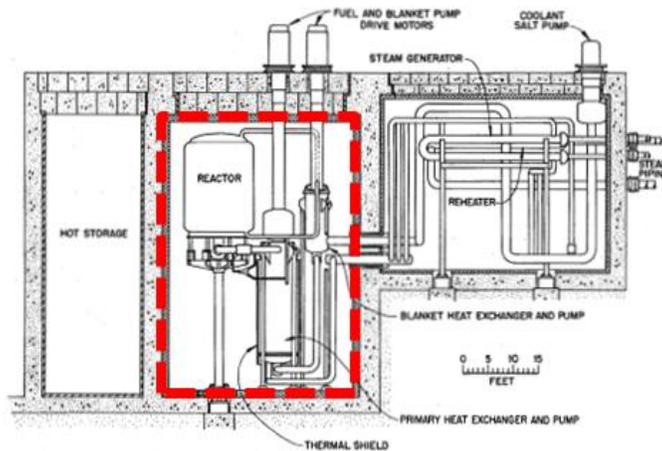


Figure 13: Elevation view of single-fluid MSR plant layout, of Oak Ridge National Laboratory (Robertson R. C., 1971).

The volume of this structure is significantly increased due to the large length of the PHX, which nearly doubles the depth of the cell. If heat exchanger modules could be placed on the same elevation as the reactor, the depth could be reduced by at least 50%.

Separating the heat exchanger load to several banks, or “modules,” of heat exchangers can provide several intriguing advantages that are not seen compared with fewer more massive heat exchangers. Whereas the ORNL design planned for four PHXs (each of which was 6.5’ in diameter and 16’ in height), smaller banks of heat exchangers can provide easier maintenance, less costly replacement, and more functional over-design. Shell-and-tube heat exchangers require removal and replacement once a certain threshold of tubes have been taken out of operation (done by “tube plugging”, where tubes are welded shut at each end once they have been deemed no long acceptable). This threshold is usually around 17% of the tubes, which leaves more than 80% of the fully-functioning tubes

The versatility gained from heat exchanger modules that can be made in any dimension (by altering the number of channels high, N_H , and number of channels wide, N_W , is another distinct advantage in MSR designs due to the importance of minimizing the volume of the secondary containment. Due to the relative ease with which molten salts solidify when not heated, the piping and components outside of the reactor core will need to be kept at elevated temperatures. The area outside of the reactor core, but still within the secondary containment, will house piping and other system components. Remote operations will be required in such an environment, but the feature is necessary to preclude any solidification of fuel or coolant salts. The cost to perform such a task will be high for a large area, so keeping the exchanger tightly wrapped around the reactor will minimize the financial burden of heating large volumes. The temperature may even be lowered if thin, wide heat exchanger modules are used in order to allow for heat to transfer to the entire salt volume within the heat exchanger. Shell-and-tube heat exchangers do not provide

the flexibility to change overall shape and would require a much larger volume to heat. While keeping material usage to a minimum is a priority for this study, three-dimensional volume usage in a plant must also be considered.

Compact heat exchangers would become increasingly attractive if the PHX was placed inside the reactor vessel. Doing so would maintain all fissile material within the reactor vessel, which would provide a robust barrier to radioactive releases. The PHX could be placed within the molten salt, but maintenance may become too difficult in the highly-radioactive environment. If an inner wall were added to the core to keep the PHX blocked from significant radiation damage and maintain it outside of the molten salt, the heat exchanger could serve the purpose of maintaining radioactive material while keeping the ability to perform maintenance and inspections.

Three-dimensional space in every direction becomes increasingly expensive inside a reactor vessel. Compact heat exchanger could provide a low-volume alternative to the shell-and-tube heat exchanger. Recent studies have shown the desire to place a heat exchanger inside a molten salt-cooled reactor (Greene, et al., 2010) but have maintained small shell-and-tube heat exchangers for the ~50 °F temperature drop used in the solid-fueled application. Heat exchangers would become much larger for the temperature drops in excess of 200 °F in MSR designs, and a movement to more compact designs may be necessary.

Figure 14 shows a simplistic diagram of the hot legs and hot channels (shown in red) and cold channel and cold legs (shown in blue) aligned at the inner wall of the containment. The heat exchanger modules, shown in black in the diagram, are arranged along the wall of the reactor containment building, which is expected to act as a “hot cell”. The secondary fluid will enter each module from outside of the containment at the bottom of the module length, will flow upward through the exchanger, and will leave containment in the same direction it arrived. The primary fluid, on the other hand, will flow out of the reactor, into a header at the top of each exchanger section, will move to a common header at a corner on each cold leg, and will then flow back into the reactor. Common headers will connect the modules on each wall (three modules on each wall are shown in Figure 14) but are not shown in the schematic below.

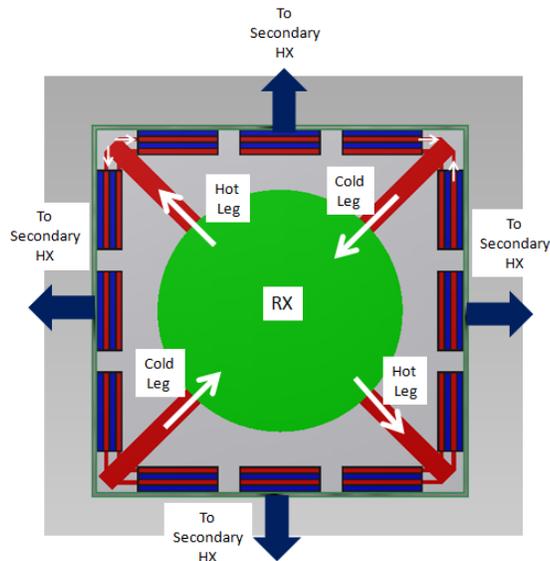


Figure 14: Sample heat exchanger layout, where modules are attached to inner wall of primary containment.

Figure 14 shows one concept for the plant set-up on a modular bank of heat exchanger modules, but several different options exist. Heat exchanger modules could closely wrap around the reactor core and fasten to the outer wall of the primary containment structure, following a circular enclosure rather than square as shown in Figure 14. As piping and space would be minimized, neutron damage effects and difficulty of manufacturing curved modules would provide downfalls. ORNL overcame this problem by inserting a thermal shield on the side of the heat exchanger closest to the reactor, which is shown in Figure 12 above. Such versatility could provide an intriguing advantage to plant designers and utilities. Minimizing the volume of the structure around the reactor core, which would be easily attained by fitting thin modules along the wall rather than a bulky circular arrangement next to the reactor, would be yet another advantage.

The set up within a plant, including header design and HX location, must be considered in a complete design. The goal of this study was to consider, not only the mechanical aspects of the heat exchanger design, but all aspects that affect the implementation of a heat exchanger in a commercial application.

Manufacturing

Fabrication and manufacturing practices play a large role in the cost of shell-and-tube heat exchangers and were considered heavily in new compact designs. Compact heat exchangers require significantly different manufacturing practices than shell-and-tube exchangers, so a new process must be developed for the heat exchanger planned in this thesis. The slotted heat exchanger design will roughly resemble a Plate Heat Exchanger (PHE), but the temperature

range and corrosive environment required would not allow a Gasketed PHE or Brazed PHE to successfully operate. Table 6 shows the range of abilities for these types of heat exchangers, which would obviously not be sufficient for the PHX of a MSR (Wang, Sunden, & Manglik, 2007).

Table 6: Comparison of gasketed plate heat exchanger with brazed plate heat exchanger.

| | Gasketed PHE | Brazed PHE |
|-------------------------------|---|------------------------|
| Maximum operating pressure | 25 bar (30 bar with special construction) | 30 bar |
| Maximum operating temperature | 160 °C | 225 °C |
| Heat transfer area | 0.1-2200 m ² | 0.02-60 m ² |
| Maximum connection size | 450 mm | 100 mm |
| Number of plates | Up to 700 | |
| Plate thickness | 0.4-1.2 mm | |
| Plate size | 0.3-3.5 m length | |

A casting process has been planned for this heat exchanger, mostly due to the simplistic nature of the geometry and more efficient use of material than milling would provide. Removing nearly all locations of welding, diffusion bonding, and gasketing would provide a much more robust design to the high-temperature, corrosive environment that the PHX will see. The ability to cast the heat exchanger structure in this study stems from the fact that the scale was brought up to a level that will not require complex processes (such as chemical etching), surface corrugations were removed to allow for straight channel lengths that can be accommodated by casting, and the monolithic structure will not require a great deal of diffusion bonding or welding. If necessary, the heat exchanger can be cast into several plates and diffusion bonded together along their length.

Casting involves creating a three-dimensional shape to create an area in which the melted metal can form around and create its shape (Rao, 2007). Several casting techniques exist, each of which has its own issues and advantages, but the details of such processes will be omitted for this study. Future research can provide relevant results in showing the ability to cast Hastelloy N, and other Nickel-based alloys, on a very small scale. Pump components were cast in Hastelloy N during the MSRE (Kasten, Bettis, & Robertson, 1966), but not nearly at the small scale required for the slotted design in this study.

Casting the length of the heat exchanger module as a monolithic structure will limit the amount of welds in the heat exchanger, which is a huge advantage due to the removal of weak points present in shell-and-tube heat exchangers. Since total heat exchanger modules may exceed the

size limit for a single casting procedure, diffusion bonding may be required for combining separate heat exchanger sections along their length.

Headers are designed to be integral to exchanger sections, allowing for monolithic structures to be cast for the entirety of the channel lengths. Common headers at the top and bottom of each heat exchanger section will lead to main plenums for each fluid, with each gathering to one side of the reactor, and is then directed toward an outlet pipe. The outlet pipe will be gasketed to allow for removal when maintenance is necessary. Rather than removing the header, this allows for the critical components that separate fuel salt from inert salt to be unaffected.

Future studies using ProCast, a commercial software package used in determining the viability of casting procedures for geometries of interest, will ultimately determine the viability of such a design.

Materials

Several choices exist for three materials decisions in such a heat exchanger design: heat exchanger metal, fuel salt, and coolant salt. The studies conducted at ORNL used a wide variety of materials for each function, but the focus of the studies was on a Hastelloy N (heat exchanger solid), F-Li-Be- UF_4 - ThF_4 (fuel salt), and $NaBF_4$ -NaF (coolant salt). Explanations of each material and the choice made for this study are shown below.

Heat Exchanger Solid

There was a relatively great deal of flexibility with the choice of HX solid, and the decision was made based on corrosion properties, heat exchanger performance capabilities, and ease of manufacturing. Hastelloy N, TZM (a Titanium-Zirconium-Molybdenum alloy), and pure Molybdenum were the final three candidates. Ceramic compounds are an intriguing option for high-temperature applications in the future, but their performance has been uninspiring thus far (Olson, 2009). Several studies have been conducted solely on this issue, so it will not be a major focus of this report.

Several of the solid materials offer intriguing benefits, but also come with potentially serious flaws. All three of the candidate solids offer exceptional corrosion resistance, especially when in contact with molten salts, but the experience with Hastelloy N provides the most appealing basis of studies regarding creep, irradiation damage, and manufacturing experience. Pure Molybdenum provides the best heat transfer properties, but it is known to be difficult to manufacture on the scale required for this design. TZM is an intriguing balance between Hastelloy N and Molybdenum, but the minor advantages it may bring in heat transfer and high-temperature

performance are outweighed by the vast experience Hastelloy N gained during ORNL's research in molten salts.

Appendix E: Material Properties

Appendix E contains a data sheet from the manufacturer of Hastelloy N, Haynes International, which provides physical properties and approximate chemical composition of the material. Concise documentation is available from ORNL during early studies looking into the requirements for MSBR materials and later development of INOR-8, which came to be known as Hastelloy N (McCoy Jr & Weir Jr, 1967).

Fuel Salt

The fuel salt is a combination of ${}^7_3\text{LiF}-\text{BeF}_2-{}^{233}_{92}\text{UF}_4$ (68.5 – 31.1 – 0.2 mole %), which is rather defined in MSBR designs and was not changed for the purposes of this study (Robertson, Smith, Briggs, & Bettis, 1968). Polynomial expressions were available for the density, thermal conductivity, and viscosity of the fuel salt (Cantor, Cooke, Dworkin, Robbins, Thoma, & Watson, 1968), while specific heat was assumed to be constant (as a temperature-dependent polynomial expression is unknown).

Appendix E contains data from ORNL regarding the material properties for the fuel salt.

Coolant Salt

The secondary coolant salt options include FLiNaK, NaBF_4 -NaF, FLiBe, and some inexpensive chloride salts that could all be used to transfer heat. Coolant salts were contrasted based on melting temperature, compatibility with Hastelloy N, cost, heat transfer characteristics, and operational experience.

Clean FLiBe (${}^7_3\text{LiF}-\text{BeF}_2$ at 66-34 mole %) was chosen due to the wealth of knowledge and experience associated with it, as well as its known compatibility similarities with the fuel salt and metals. FLiBe is typically enriched in lithium-7 to enhance neutronics, but this costly procedure may be avoided in secondary salts to provide a more cost effective solution. Cheaper options exist for the coolant salt, but changing the composition will not affect results greatly due to the relative similarity of all molten salts' properties. The wealth of knowledge and experience about FLiBe was an extremely attractive option for the secondary salt, and was weighed more heavily than cost, which is minimal when considered relative to the scale of the entire plant (Robertson, Smith, Briggs, & Bettis, 1968).

Polynomial expressions were available for the density, thermal conductivity, and viscosity of FLiBe (Sohal, Sabharwall, Sharpe, & Ebner, 2010), while specific heat was assumed to be constant (as a temperature-dependent polynomial expression is unknown).

Appendix E contains the fluid properties of the $NaBF_4$ -NaF used in the MSBR design as well as those for FLiBe used in this study.

Heat Exchanger Design

Compact heat exchangers have evolved greatly in the last several decades to become more compact and efficient than ever. Modern materials advancements and manufacturing techniques have allowed for dimensions to steadily decrease, resulting in designs that are increasingly more compact and efficient. The use of circular, or semi-circular, channels has been utilized in several modern designs with channel dimensions much less than 1mm in hydraulic diameter, with performance that reflects the complexity of the designs. Though those dimensions are not reasonable for the purposes of a PHX, mostly due to maintenance restrictions (channels must be large enough to perform reasonable inspections), the concept of using smaller channels remains an interesting concept. The degree to which the PHX dimensions can ultimately approach those of modern compact heat exchangers will ultimately determine the viability of compact heat exchangers in the nuclear industry.

Removing and/or altering several characteristics of recent compact heat exchangers, such as the Printed Circuit Heat Exchanger (PCHE) designed by Heatric, can result in a heat exchanger design suitable for use in a Molten Salt Reactor. The concepts that make the design of the heat exchanger in this study innovative are explained in the subsequent section.

Design Changes

The two most prominent heat exchanger designs for uses in nuclear applications, albeit on the secondary molten-salt-to-gas heat exchanger, are the PCHE (Figley, 2009) and Offset Strip-Fin HX (Hechanova, 2007). These heat exchanger designs employ channels on the micro-scale by using complex chemical etching and diffusion bonding processes to construct the heat exchanger. Maintaining such designs would become very difficult as channel dimensions grow smaller and surface features become more complex, so several aspects of these designs must be stripped away in order to arrive at a design that can be adequately maintained.

The PCHE and Offset Strip-Fin HX each have channel diameters well below 1mm, which lead to excellent efficiencies and can provide compactnesses of up to $2500 \text{ m}^2/\text{m}^3$. Though that scale provides several difficulties in molten-salt-related applications, the potential rewards are significant. Fouling could easily block channels on such a small scale, and mal-distribution effects would become even stronger in individual channels where that occurred. Loose impediments present in any flow path in complex nuclear applications could also provide significant detriments to performance in smaller, individual channels more so than in other

designs. Loose parts, solids that have plated out during operation, and other impediments are inherently present in plants during operation, so precautions such as filters and screens must be provided. Raising channel dimensions, namely height, to become less susceptible to blockage can further increase robustness.

As previously described, high-frequency eddy current inspections have been used in medical applications on circular channels of very small dimensions, but the smallest size to date was only 1.8034 mm in diameter. This provides an approximate lower limit of the channel dimensions, but also provides an important maintenance advantage for this design. In order to push the limits of heat exchanger performance, this dimension must be further reduced in full-scale final design options. The development of eddy current probes of even higher frequency may be required to meet performance requirements. Transitioning from more well-known circular cross-section eddy current probes to pancake-style probes will also be a required design challenge.

The design presented in this report will feature high-aspect-ratio rectangular channels on the scale of 0.5-5 mm in height, with resulting hydraulic diameters in the range of 1-5 mm. “Slotted” channels can be limited in flow velocity to maintain laminar flow, especially when compared to channels of circular cross-section. The lower flow velocities and increased ratio of area-to-perimeter tend to hurt heat transfer performance and raise pressure drop, but their efficient use of material can justify small decreases in heat transfer performance.

Recent manifestations of compact heat exchangers in nuclear applications, such as the PCHE or Offset Strip-Fin HX, use circular or semi-circular channel cross-sections and are readily produced by revolutionary chemical etching and diffusion bonding processes. The scale at which they are produced limits the cross-section to these shapes, but larger channels would not require such complex processes. A transition to a simple casting process could provide attractive simplifications when compared to compact heat exchanger processes and economic advantages compared to shell-and-tube processes.

In order to further simplify manufacturing to a traditional casting process, as well as to provide more well-known maintenance options for the heat exchanger, surface corrugations and wavy channel paths have been omitted from the design. These design options increase heat transfer performance and limit pressure drop, but render the heat exchanger channels difficult to maintain and produce. If maintenance techniques are proven to handle such surface features in the future, corrugations could then be added to the design to improve its performance. Relatively simple chevron patterns may be added if eddy current probes can follow the path down the length of the exchanger, but preliminary designs will omit such features at the detriment of their final results.

The ultimate heat transfer performance would be seen from heat exchanger channels of alternating composition in each direction, what can be called a “checkerboard” configuration.

Header designs for heat exchangers of alternating composition in each direction are prohibitively complex and difficult to maintain. For the simplicity of header designs, each row of the heat exchanger was kept with the same fluid. The header can then be simplified to an extension of the channel length, with each fluid being brought together on opposing sides of the heat exchanger. Headers can be cast, integral to the rest of the heat exchanger length, as a further means for simplifying and streamlining the manufacturing process. Figure 15 shows a simple sketch of such a header design, where the “hot” fluid (red) is shown flowing into the “hot” header from the left and downward through the heat exchanger and the “cold” fluid (blue) is shown flowing upward through the channels and out of the “cold” header to the right:

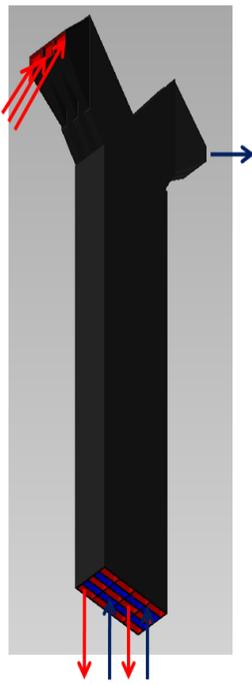


Figure 15: Slotted minichannel heat exchanger with header appearing as a continuation of the channel length.

With rows using the same fluid across their entirety, it no longer made sense to contain the fluid region into a larger number of separate circular channels. The use of large, thin openings of rectangular cross-section could provide an avenue to lower the amount of material used in the design while accommodating similar usable heat transfer area. This function removes unnecessary material between adjoining channels of similar composition, thus raising the material efficiency (SA/MAT) without damaging heat transfer performance. The subsequent section of this report will discuss the performance of heat exchangers utilizing channels of circular cross-section and the “slotted” design.

Slotted Design

Figure 16 shows an isometric view of a 2-channel by 4-channel array used in introductory studies for the slotted heat exchanger. The 8 channels, which act as a 4-channel by 4-channel array (16 total channels) due to the symmetry plane shown on the right side of the drawing, will be used in preliminary studies to compare channel dimensions and provide validation data.

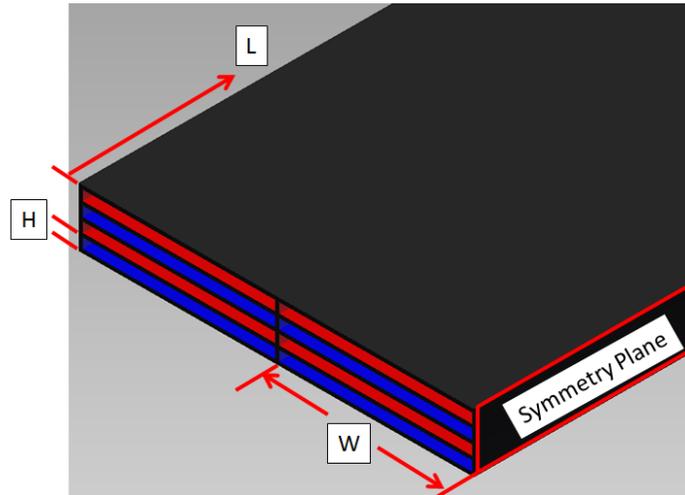


Figure 16: 2-channel by 4-channel array of slotted channel used in preliminary study.

The diagram shown in Figure 16 displays the entirety of the geometric parameters for a theoretical “slotted” heat exchanger design containing 16 channels. Channel height (H), channel width (W), and channel length (L) were altered to determine the optimal design. The spacing between channels, both in the height-direction (SH) and width-direction (SW), were manipulated to provide sufficient strength. Channel dimensions were parameterized in ANSYS Workbench to allow for easy manipulation of each of the design’s input parameters.

The channel height (H) will extend radially outward from the reactor, with the number of channels in height, N_H , controlling the radial dimension of the heat exchanger module as they are connected along their length to an outer containment wall parallel to the height of the reactor. The heat exchanger will be arranged around the reactor, with the Length (L) pointing vertically. The heavier fuel salt will be pumped downward through half of the heat exchanger channels, while the coolant salt will flow upward through the remaining channels. As shown in Figure 14, the fuel salt will return to the reactor on the two cold legs of the reactor core.

Full-scale studies will use a larger array of channels but will employ a similar method of construction. Whereas 16 channels are modeled in the preliminary studies, 45 channels will be studied in the full-scale simulation. An array, 2.5 channels in width and 4.5 channels in height, will be modeled as shown in Figure 17. The symmetry plane (shown in yellow) was placed in the

center of channels in both the width- and height-directions to allow for greater returns from the symmetry condition.

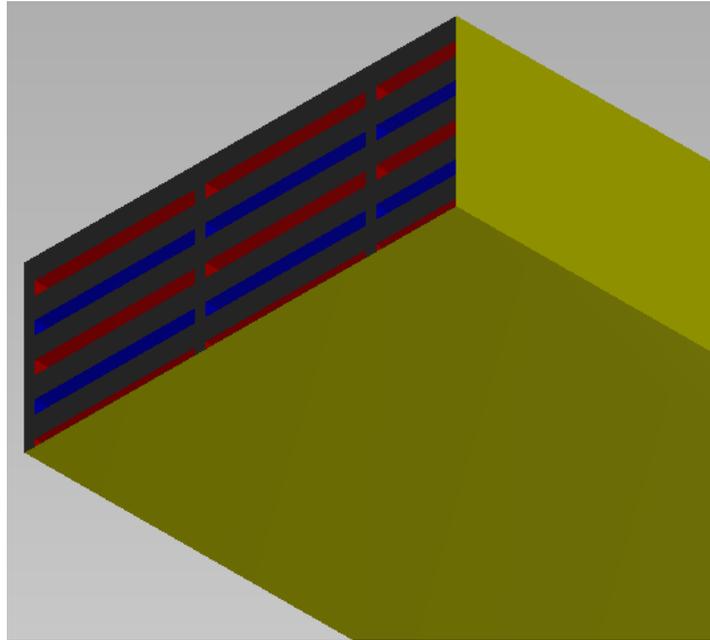


Figure 17: Full-size channel array of 2.5 channels by 4.5 channels (hot channels shown in red, cold channel shown in blue, symmetry plane shown in yellow, and HX material shown in black).

Compact heat exchanger channels of rectangular cross-section are not a revolutionary concept by themselves, but finding an intermediate range of high aspect ratio rectangular minichannels in a MSR application differs greatly from the original design. With a known concept that is expected to provide reasonable returns in performance, a slotted heat exchanger must be modeled and simulated to confirm its results. The subsequent several sections will lay out the process used to model heat transfer in the slotted minichannel system.

Assumptions

Several characteristics of the design can be simplified or ignored to allow for more manageable computations in the early stages of design.

Symmetry

The computational domain of a heat exchanger can be quartered by dividing the geometry into four equal sections. Results can be extended to the other three sections to gain total results, but the ability to lessen computational requirements by more than 75% is a huge advantage when limited computing power is available. Figure 18 shows the computational domain being cut in half by dividing the geometry into two equal halves. As long as boundary conditions and

geometry are indeed symmetric on each side of the division (shown in red), the hashed yellow portion of the sketch no longer needs to be modeled.

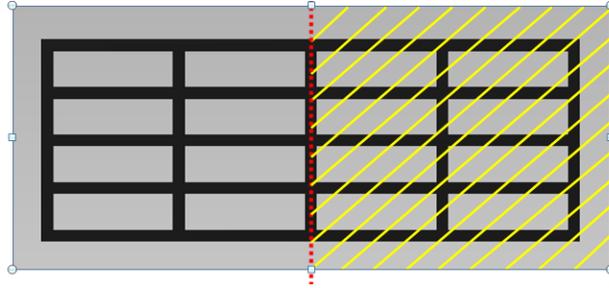


Figure 18: Vertical symmetry plane that halves computational domain.

Figure 19 shows a similar ability to halve the domain in each direction, with the vertical symmetry prescribed in this case. Again, boundary conditions and geometry must be perfectly symmetric on each side of the symmetry line (shown in red) in order for the physics to be accurately captured.

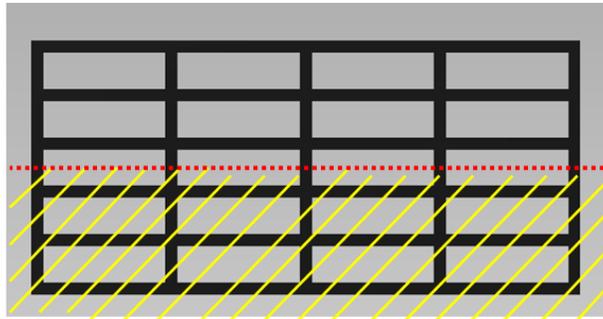


Figure 19: Horizontal symmetry plane that halves computational domain.

The ability to quarter the computational domain, thus reducing the computational load by more than 75%, is a significant advantage when larger array sizes are required for full-scale simulations. The symmetry of the physics and geometry demonstrated in Figures 18 and 19 shows the viability of using only a quarter of the geometry in FLUENT. Simple “Symmetry” boundary conditions are assigned to the correct planes (the two intersecting dividing planes shown in red in each sketch), allowing FLUENT to duplicate the results on each side of the plane.

Perfect Casting

The model used in this study has perfectly square channels to allow for simpler and easier meshing. In reality, parts will come out with corners that appear more as fillets than as right angles. The effects of such geometry changes were assumed to be negligible in this study but should be addressed in more concise studies in the future. Figure 20 shows how a rectangular channel would actually appear, with fillets in the corners resulting from casting, while the perfect geometry used in FLUENT is shown on the right.



Figure 20: Actual manufactured channel (left) will differ from modeled channel (right) in applications.

Assuming the satisfaction of negligible manufacturing tolerances is generally acceptable on a small scale but compounding the error of one channel on *thousands* of channels will result in measurable changes. However, any curved edges will only improve the performance because they will eliminate the no-heat-flux corners that hurt the performance of rectangular channels.

Corrosion/Erosion Effects Negligible

The effects of corrosion and erosion will be considered throughout the design of the heat exchanger, but fouling effects cannot be properly modeled in FLUENT. Conservative design ranges should be considered to allow for the increased effects of corrosion and erosion as operation time lengthens.

Channels of insufficient dimension could be blocked by fouling, rendering them useless. Those that could still remain in operation with corrosion effects could result in hot spots and/or uneven wear due to the extended surface. Heat transfer performance will be hindered in all cases. However, the model used in this research assumes that corrosion and erosion effects are negligible. Future studies with physical test apparatuses can determine the actual changes.

Steady-state Operation

Transient conditions must be tested separately from the normal operating conditions of the heat exchanger due to the important effects of thermal transients on compact heat exchangers. This study, however, will focus on steady-state conditions.

Single-Phase Flow

Molten salts have high boiling temperatures that will not be approached in the temperature range used in this study. FLiNaK, a common molten salt considered to be a candidate secondary cooling salt, for example, has a boiling point of 1843 K (Sohal, Sabharwall, Sharpe, & Ebner, 2010). By maintaining temperatures below 1000 °C, single-phase flow can be assumed through calculations.

Outer Wall of Heat Exchanger Boundary Condition

All piping and heat exchangers must be placed in an elevated temperature inside primary containment just outside of the reactor vessel in order to disallow the possibility of solidification of the coolant. For these reasons, heat flux on the outer surfaces of the wall has been assumed to be 0 in many studies (Figley, 2009). Such a boundary condition will not properly convey the physics occurring in this application.

The construction of a Molten Salt Reactor will require assurance that the salt will never solidify in the piping and heat exchanger. The addition of clamshell heaters surrounding every pipe and component would be costly, cumbersome, and would bring forth significant hurdles in maintenance operations. The MSBR planned for all necessary components to be contained in a secondary environment raised to a temperature of 1100 °F (866.5 K) to ensure that the coolant, blanket, and fuel salts will remain in their liquid forms even when the reactor system temperature drops nearer their melting points (Kasten, Bettis, & Robertson, 1966).

For this study, outer walls of each heat exchanger array were assumed to be at the elevated room temperature of 1100 °F.

Roughness Effects

For the purposes of this study, roughness along the channels was assumed to be negligible. Casting and manufacturing processes will never present a surface that is without some minor roughness effect, but the increased computational load required to model such an effect is not necessary for the scope of this research. Any roughness actually present in a manufactured and tested heat exchanger will enhance heat transfer performance.

Inlet/Outlet Effects

Headers have been neglected in the work performed in this report. For this reason, heat exchange in the cross-flow region of the headers will be neglected. Hydrodynamic effects, such as sudden expansion and contraction of flow area, will also be neglected in this study.

Composition-dependent Salt Properties

A Fluorine-Lithium-Beryllium compound composed of ${}^7\text{LiF-BeF}_2$ (66-34 mole %) was developed and studied in-depth at ORNL in the 1960's and 1970's. The properties and behavior of this fluid was chosen for use in the MSR, and a great deal of knowledge was developed in the decades of its prominence. In 1967, ORNL published a report based solely on describing the properties of FLiBe, while providing temperature-dependent correlations for each of its major heat transfer properties (Cantor, Cooke, Dworkin, Robbins, Thoma, & Watson, 1968).

The coolant salt, assumed to be FLiBe for this study, will have a relatively similar composition to that of the fuel salt. Comprised mostly of the same constituents as FLiBe, with trace amounts of Uranium (U) added, the fuel salt will inherently have dissimilar properties to the clean secondary salt. The degree to which the two liquids' properties are different was not known, and was studied to assure that they must be modeled as such.

Equations 1 through 4 describe the fluid properties for FLiBe (Cantor, Cooke, Dworkin, Robbins, Thoma, & Watson, 1968):

$$\mu = 0.116 \exp\left(\frac{3755}{T[\text{K}]}\right) [\text{cP}] \quad \text{Equation 1}$$

$$k = 0.01 \left[\frac{\text{W}}{\text{cm}^\circ\text{C}}\right] \quad \text{Equation 2}$$

$$C_p = 0.57 \left[\frac{\text{Cal}}{\text{g}^\circ\text{C}}\right] \quad \text{Equation 3}$$

$$\rho = 2.214 - 4.2 * 10^{-4}T[^\circ\text{C}] \left[\frac{\text{g}}{\text{cm}^3}\right] \quad \text{Equation 4}$$

ORNL released a document regarding the properties of several salts, with the Thorium- and Uranium-containing fuel salt being of importance for this study in the same 1968 document (Cantor, Cooke, Dworkin, Robbins, Thoma, & Watson, 1968). The correlations from this source for fuel salt are laid out in Equations 5 through 8.

$$\mu = 0.084 \exp\frac{4340}{T[\text{K}]} [\text{cP}] \quad \text{Equation 5}$$

$$\mathbf{k} = \mathbf{0.01} \left[\frac{\mathbf{W}}{\mathbf{cm}^* \mathbf{^{\circ}C}} \right] \quad \text{Equation 6}$$

$$\mathbf{C_p} = \mathbf{0.34} \left[\frac{\mathbf{Cal}}{\mathbf{g}^* \mathbf{^{\circ}C}} \right] \quad \text{Equation 7}$$

$$\mathbf{\rho} = \mathbf{3.628} - \mathbf{6.6} * \mathbf{10}^{-4} \mathbf{T} [^{\circ}\mathbf{C}] \left[\frac{\mathbf{g}}{\mathbf{cm}^3} \right] \quad \text{Equation 8}$$

The option of assuming similar properties for the fuel and coolant salts was explored, and recent results from Idaho National Laboratory (INL) were used after being studied extensively in 2010 (Sohal, Sabharwall, Sharpe, & Ebner, 2010). Equations 9 through 12 shows the most recent correlations for the properties of FLiBe.

$$\mathbf{\rho} = \mathbf{2413} - \mathbf{0.488T} [\mathbf{K}] \left[\frac{\mathbf{kg}}{\mathbf{m}^3} \right] \quad \text{Equation 9}$$

$$\mathbf{\mu} = \mathbf{0.000116} \exp \left(\frac{\mathbf{3755}}{\mathbf{T} [\mathbf{K}]} \right) [\mathbf{Pa} - \mathbf{s}] \quad \text{Equation 10}$$

$$\mathbf{C_p} \cong \mathbf{2350} \left[\frac{\mathbf{J}}{\mathbf{kgK}} \right] \quad \text{Equation 11}$$

$$\mathbf{k} = \mathbf{0.629697} + \mathbf{0.0005T} [\mathbf{K}] \left[\frac{\mathbf{W}}{\mathbf{mK}} \right] \quad \text{Equation 12}$$

The measurement techniques used in the 2010 INL experimentation are much more precise than those used in the 1960's when ORNL recorded the values for fuel salt. Recent studies involving clean molten salts have increased the knowledge of those fluids, but fuel salts have been ignored since the MSRE in the 1960's and 1970's. For this reason, the fluid properties for the fuel salt given in ORNL documentation (Kasten, Bettis, & Robertson, 1966) are the only option for contrasting against the clean salt. Table 7 has the fluid properties for fuel salt, given with relatively large tolerances due to the limitations brought on by measuring equipment.

Table 7 shows a preliminary study on the FLiBe properties found in the studies in 1968, where the composition was altered slightly to account for some discrepancies. The correlations were applied at the anticipated average fluid temperature of 1150 °F.

Table 7: Properties of FLiBe from old study (ORNL) and recent study (INL).

| Document | ORNL 2316 | INL – 18297 | |
|-----------------------------|-----------|-------------|--------------|
| Year | 1968 | 2010 | |
| Composition | LiF-BeF2 | LiF-BeF2 | |
| Composition % | 66-34 | 67-33 | |
| T_{ref} [F] | 1150 | 1150 | |
| T_{ref} [C] | 621.11 | 621.11 | |
| T_{ref} [K] | 894.26 | 894.26 | % difference |
| μ [Pa-s] | 0.0077279 | 0.0077279 | 0 |
| C_p [J/kg-K] | 2386.5 | 2365 | 0.9090909 |
| k [W/m-K] | 1 | 1.076827 | 7.1345722 |
| ρ [kg/m ³] | 1953.1338 | 1976.77 | 1.195698 |

Table 8 shows the results comparing the best-known correlations for fuel salt at 1150 °F with the best-known results for FLiBe at the same temperature.

Table 8: Comparison of FLiBe with fuel salt.

| Document | ORNL 2316 | INL - 18297 | |
|-----------------------------|-------------------|-------------|--------------|
| Year | 1970 | 2010 | |
| Composition | LiF-BeF2-ThF4-UF4 | LiF-BeF2 | |
| Composition % | 73-16-10.7-0.3 | 67-33 | |
| T_{ref} [F] | 1150 | 1150 | |
| T_{ref} [C] | 621.11 | 621.11 | |
| T_{ref} [K] | 894.26 | 894.26 | % difference |
| μ [Pa-s] | 0.010764312 | 0.0077279 | 28.208138 |
| C_p [J/kg-K] | 1423.5 | 2365 | 66.139796 |
| k [W/m-K] | 1 | 1.076827 | 7.1345722 |
| ρ [kg/m ³] | 3218.07 | 1976.77 | 38.572809 |

Temperature-dependent Salt Properties

The relatively wide range of temperatures used in this study led to material properties that were expected to vary rather significantly. A preliminary study on the temperature-dependence of FLiBe at each end of the temperature spectrum was studied.

Table 9 shows the change in fluid properties of FLiBe across the secondary coolant’s temperature range (850-1111 °F). Significant changes, especially in viscosity, are seen for FLiBe.

Table 9: Properties of FLiBe over its operating range.

| Document | INL – 18297 | INL – 18297 | |
|-----------------------------|-------------|-------------|--------------|
| Year | 2010 | 2010 | |
| Composition | LiF-BeF2 | LiF-BeF2 | |
| Composition % | 67-33 | 67-33 | |
| T | Minimum | Maximum | |
| T_{ref} [F] | 850 | 1111 | |
| T_{ref} [C] | 454.44 | 599.44 | |
| T_{ref} [K] | 727.5944 | 872.59 | % difference |
| μ [Pa-s] | 0.02022016 | 0.008577103 | 57.581427 |
| C_p [J/kg-K] | 2365 | 2365 | 0 |
| k [W/m-K] | 0.993494 | 1.065994 | 6.801164 |
| ρ [kg/m ³] | 2058.555 | 1987.400672 | 3.4565182 |

Just as Table 9 studies the change of the secondary coolant in its respective temperature range, Table 10 shows the change of the fuel salt across its temperature range (1000-1300 °F). The fuel salt’s known correlations are relatively inaccurate because of the limited measuring resources at the time of their collection.

Table 10: Properties of fuel salt over its operating range.

| Document | ORNL 2316 | ORNL 2316 | |
|-----------------------------|-----------------------|-------------------|--------------|
| Year | 1970 | 1970 | |
| Composition | LiF-BeF2- ThF4-UF4 | LiF-BeF2-ThF4-UF4 | |
| Composition | 73-16-10.7- | | |
| % | 0.3 | 73-16-10.7-0.3 | |
| T | Minimum | Maximum | |
| T_{ref} [F] | 1000 | 1300 | |
| T_{ref} [C] | 537.78 | 704.4 | |
| T_{ref} [K] | 810.93 | 977.59 | % difference |
| μ [Pa-s] | 0.007117457 | 0.01772438 | 149.02686 |
| C_p [J/kg-K] | 1423.5 | 1423.5 | 0 |
| k [W/m-K] | 1 | 1 | 0 |
| ρ [kg/m ³] | 3092.78 | 2982.79 | 3.5563474 |

The results showed that the clean FLiBe and fuel salt were quite different in their material properties, and it was necessary that they be modeled and treated as such in FLUENT. In addition to differentiating between fuel salt and secondary salt, the results showed that properties varied widely over the temperature range in which they operate. For this reason, polynomial expressions were found for each material property for later use in FLUENT.

In order to be able to use the “Polynomial” function for temperature-dependent properties in FLUENT for the FLiBe density function, which is given as an exponential in Equation 13, the Curve Fitting Toolbox was used in MatLab to match the exponential equation to an equal polynomial over the same price range. With an R-value of exactly 1, the following second-order polynomial equation is a perfect match between 800K and 1100K:

$$\mu = -1.792E^{-10}T^3 + 5.922E^{-7}T^2 - 0.0006654T + 0.2573 \quad \text{Equation 13}$$

where μ is the fluid viscosity and T is the fluid temperature, measured in Kelvin. Fluid properties varied widely in both fuel compositions and across the temperature range, so polynomial expressions of each composition were used in FLUENT during this study. A simplification of common materials and constant material properties would have rendered the FLUENT model inaccurate, and adding each respective aspect of the design was not prohibitively time-consuming.

These findings and decisions will play a large role in determining the viability of the CFD simulations carried out in this research. The FLUENT design process will be laid out in the next section of this thesis.

FLUENT Design Process

ANSYS Workbench, in conjunction with FLUENT and several built-in design modules, was used to build, design, analyze, and optimize the design of the slotted heat exchanger in this study. Workbench displays the steps needed throughout the process, and each module will be described as it appears in Figure 21.

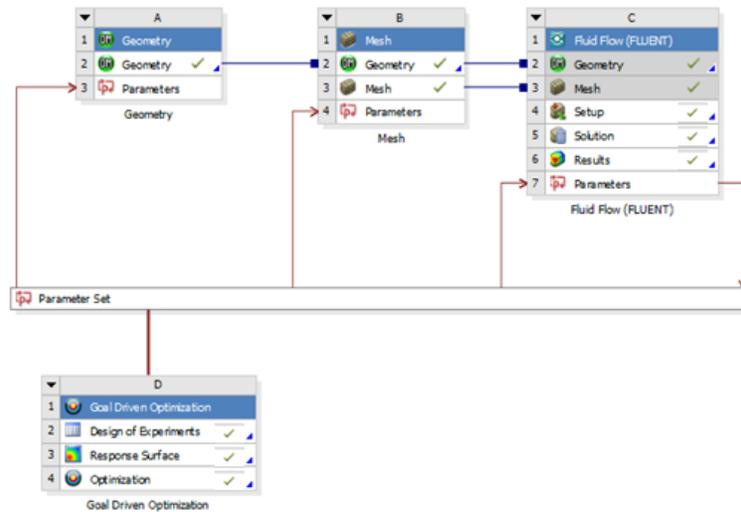


Figure 21: ANSYS Workbench interface showing FLUENT and Goal-Driven Optimization modules.

The subsequent sections will cover the theory and process followed for each step, as performed in ANSYS Workbench.

Geometry

The first step in preparing a problem for ANSYS/FLUENT is creating and defining the cell zone domains for later use in the problem. ANSYS DesignModeler is the geometry builder in ANSYS Workbench that was used to construct the heat exchanger solid regions and fluid regions in this project.

The model was built with the ability to parameterize each dimension for ease of change later. Parameterizing each dimension also allowed for automated altering of dimensions, as described later in this section.

First, the solid section of the heat exchanger was constructed in numerous individual pieces. Each piece was selectively chosen to allow for matching nodes along the solid-fluid interface and rectangular mesh distribution throughout the domain. As shown in a front view of the model in Figure 22, the different colored pieces denote separate sections created in DesignModeler.

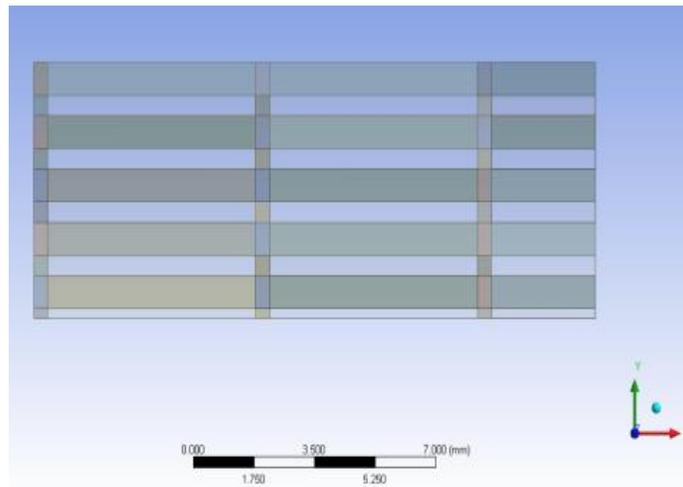


Figure 22: Full-size geometry model created in ANSYS DesignModeler with only solid regions built.

After creating the solid portion of the exchanger that surrounds the fluid in the heat exchanger, the “Fill” function was used to fill the empty space between the solids. This creates the fluid domain for each channel of the heat exchanger. The cross-section of the final geometry model appears as it is shown in Figure 23 below.

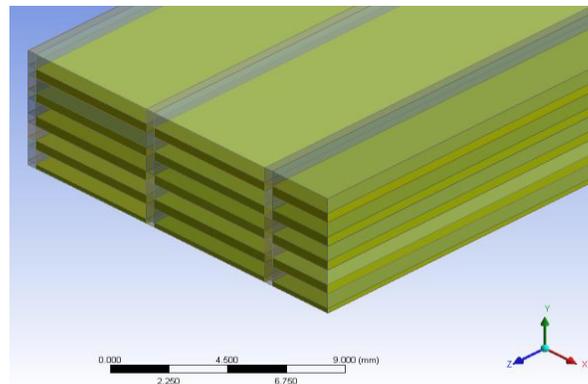


Figure 23: Full-size model from ANSYS DesignModeler with fluid regions filled and highlighted.

ANSYS DesignModeler has the ability to parameterize all dimensions to allow for automated manipulation later in the design process. In order to take advantage of this function, the dimensions were entered as seen in Figures 24 and 25.

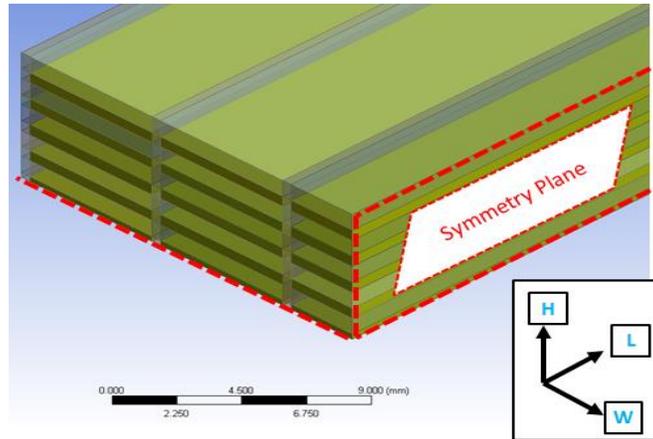


Figure 24: Isometric view of full-scale geometry with symmetry plane shown.

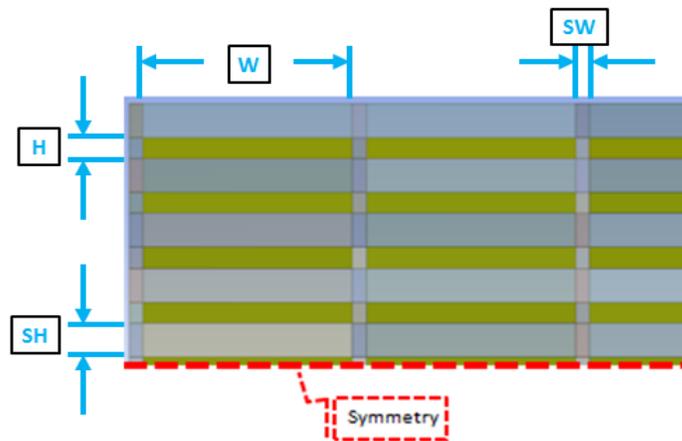


Figure 25: Front-view of full-scale geometry from ANSYS DesignModeler with geometric parameters shown.

Preliminary studies use geometries with 2 channels in the width-direction of similar composition and 4 channels stacked vertically of alternating composition. Heat transfer coefficient results for the inner channels were expected to be roughly equal to that when the heat exchanger is extended to a larger scale, so optimization was performed on a smaller geometry to decrease run times. The relative performance of heat exchanger channels of different sizes and lengths were first studied on an array of channels smaller than those shown in Figures 24 and 25 in order to decrease run times.

Full-scale studies must approach the actual size of a heat exchanger module using an array of 5 modules in width and 9 modules in height. Little return is seen in larger arrays, so the maximum array size used in this study is 45 total channels (by means of a 2.5-channel x 4.5-channel quartered model).

Mesh

Meshing is the discretization of cell zones throughout the solid and fluid domains to allow for accurate control volume analyses (ANSYS, Inc., 2009). The process of creating a viable mesh is influential in producing results that are accurate. Figure 26 displays the simple, structured geometric mesh used for an initial geometry in this study.

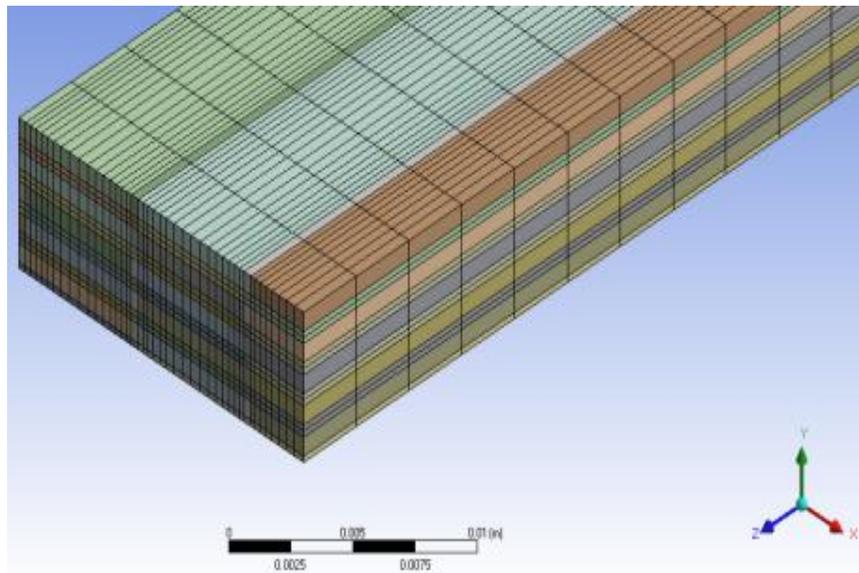


Figure 26: Mesh created for full-scale model using ANSYS Mesher.

Due to the iterative dimensioning process used in later optimization efforts through ANSYS Workbench, the mesh was built to be robust to changes. Edge sizing based on Element Size, rather than Number of Elements, was used to maintain constant mesh sizing and accuracy in larger geometries. Table 11 displays the mesh values used for all geometries.

Table 11: Mesh parameters used in full-scale model.

| | X (Width) | Y (Height) | Z (Length) |
|-------------|--------------|--------------|------------|
| Direction | | | |
| Scale | ~ 50mm | ~ 10mm | ~ 750mm |
| Sizing Type | Element Size | Element Size | Num of Div |
| Dimension | 1e-4 m | 5e-4 m | 1000 m |
| Bias | * Figure 27 | * Figure 27 | None |
| Bias Factor | 2 | 1.5 | N/A |

Figure 27 shows the biasing used to have higher mesh concentration near edges and coarser mesh in less important central areas. This change was made to ensure that boundary layer growth and behavior was accurately captured during its growth. The “inverse bell curve” mesh is a method for more economically distributing mesh elements to produce similarly correct results (Figley, 2009).



Figure 27: Mesh distribution that biases smaller nodes closer to outer walls.

Bias Factors are available in the ANSYS Mesher to account for more variations near wall boundaries than at the center of flow regions, where changes are not as prominent. The same number of nodes can be positioned more efficiently along each direction to better capture the physics occurring. As seen in Table 11, Bias Factors were used on both the fluid height and fluid width in order to capture any effects that may be seen on the boundary-layer-level in each direction. The effects of Bias Factors can be seen in Figure 26, where mesh elements are used more efficiently to cover the same length span.

The details and results of the parametric mesh study are shown in Appendix B.

Mesh Quality

Limitations of mesh density are not only based on the accuracy of the results rendered but also based on several mesh statistics that describe the geometric properties of the mesh elements. Skewness and Aspect Ratio were the two fundamental measures of mesh quality used in this study.

Skewness measures a mesh cell’s deviation from a cell with equal sides and angles throughout its volume. Table 12 shows the ranges in which skewness is measured, and the respective quality for each provided by ANSYS (ANSYS, Inc., 2009):

Table 12: Ranges of mesh cell skewness and their respective ranges.

| Value of Skewness | Cell Quality |
|-------------------|--------------|
| 1 | degenerate |
| 0.9 -- <1 | bad |
| 0.75 – 0.9 | poor |
| 0.5 – 0.75 | fair |
| 0.25 – 0.5 | good |
| >0 – 0.25 | excellent |
| 0 | equilateral |

For 3-D meshes, mesh skewness values at or below 0.4 are said to be acceptable (ANSYS, Inc., 2009). Equation 14 describes the process for finding the skewness of a cell element.

$$\text{Skewness} = \frac{\text{Optimal Cell Size} - \text{Cell Size}}{\text{Optimal Cell Size}} \quad \text{Equation 14}$$

Aspect ratio, another key metric that describes the viability of mesh elements, describes the relationship between the dimensions of an element in all three spatial directions. As the aspect ratio grows larger, the element becomes less cubic, and results will deteriorate. Finding a balance between lowering the number elements (high aspect ratio) with the optimal solution accuracy (low aspect ratio) can be found around an aspect ratio of 3:1. Two extremes, a square with an aspect ratio near 1:1 and a thin rectangle with an aspect ratio near 20:1, are shown in Figure 28 (ANSYS, Inc., 2009).

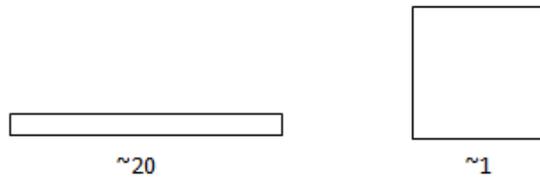


Figure 28: Two cells, one of high aspect ratio (left) and one of minimal aspect ratio (right).

All meshes were checked to determine that aspect ratio and skewness were kept to acceptable levels, as prescribed in the previous several paragraphs. Special care was taken to conservatively sacrifice mesh count, and thusly run time, in order to maintain accuracy in the results.

FLUENT Setup

Creating the geometry and mesh are major steps toward solving a CFD problem, but assigning boundary conditions and other necessary characteristics can be done in the Setup module of FLUENT within ANSYS Workbench. The important features and setup characteristics will be explained here.

Models

Several models are available in FLUENT for applications requiring complex physical processes. Obscure models, such as Radiation and Acoustics, were kept “Off”, as this simplistic model does not require such capabilities. Table 13 summarizes the models used in the FLUENT model in this study.

Table 13: Models used in FLUENT set up of full-scale simulation.

| Model | Status (On/Off/Other) |
|--------------------------|------------------------------|
| Multiphase | Off |
| Energy | On |
| Viscous | Laminar |
| Radiation | Off |
| Heat Exchanger | Off |
| Species | Off |
| Discrete Phase | Off |
| Solidification & Melting | Off |
| Acoustics | Off |

Due to compact heat exchangers nearly always being operated in the laminar regime, in order to maintain reasonable pressure drops, preliminary studies were performed using the Laminar model. Future studies may require improved heat transfer, so subsequent iterations of this study may replace this with the Turbulent model.

Materials

Material property correlations were presented in the previous section of this report, and the reasoning for providing all material decisions was also described. FLiBe was chosen as the secondary salt, uranium-containing FLiBe was chosen as the fuel salt, and Hastelloy N was chosen as the solid material. The temperature-dependent polynomial expressions provided for each material in ORNL documentation, as described previously, were entered into FLUENT Set Up.

Cell Zone Conditions

As FLUENT reads the mesh prepared in ANSYS Mesher, the Cell Zones conditions read in the separate regions of solid and fluid. Due to the geometry and mesh prepared previously to FLUENT reading the data, a separate (N) number of contiguous fluid regions are imported and surrounded by one monolithic solid structure.

Boundary Conditions

Several boundary conditions were input at inlets, outlets, and outer surfaces to arrive at the appropriate physics that such a heat exchanger would see in practical use. Some constraints were given from ORNL specifications and others provided some freedom in changing parameters to arrive at an optimal design.

Velocity inlets were prescribed at each inlet face for both fluids in all studies. As mentioned previously, the velocity must assure laminar flow throughout the heat exchanger. The velocity inputs for each fluid were parameterized to allow for automated manipulation in ANSYS Workbench. Inlet temperatures were strictly kept to the values given by ORNL, where the fuel salt entered at 1300 °F to be cooled and the secondary coolant salt entered at 850°F to be heated. These were strict boundary conditions followed by all ORNL heat exchanger designs, so they were used similarly in this design.

The outlet faces for each fluid region were parameterized to allow for a change from 0 gauge pressure, which simply measures the pressure drop for a given arrangement, to higher pressures that more accurately mirror the design conditions seen in ORNL designs. With ANSYS Workbench able to manipulate such boundary conditions, a seamless transition from small-scale to full-scale is possible.

In locations where fluid cell zones come into direct contact with solid cell zones, FLUENT automatically creates an interface area called a “shadow zone” that allows for a more appropriate boundary condition to be applied to the surface. In order to allow for conjugate heat transfer, shadow zones allow for a “Coupled” thermal boundary condition that recognizes that no other boundary condition must be prescribed at such a location.

On the outer surface of the solid, a constant temperature boundary condition was entered as an “Input Parameter” that can be manipulated in ANSYS Workbench. The heat exchanger will be kept in a heated environment outside out primary containment in order to avoid solidification of either salt. The temperature at which the room must be maintained was succinctly described in ORNL paperwork as 1100°F. Higher temperatures lead to more power spent heating the room, but changing the outer surface temperature could provide an intriguing change to the physics that may lead to improved performance. Future studies could inclusion more in-depth focus on changing the outer wall temperature, as thin arrays of heat exchanger banks could provide an easier means for avoiding solidification of the salts.

As explained previously in the Assumptions section of this report, symmetry boundary conditions were placed at several locations to allow for only a fraction of the computational load to accurately model the physics of the system. Lessening the number of cells will make runs much more manageable

The remaining interior walls and surfaces do not require any boundary conditions to accurately capture the heat transfer and hydrodynamics processes occurring in the flow channels.

Solution

Once properly set up and initialized, the discretized domains can be solved for the appropriate governing equations to provide velocity, pressure, and temperature profiles. With those results known, heat transfer and hydrodynamic properties can be extracted as needed.

FLUENT goes about solving fluid flow by discretizing the domain into separate control volumes (Meshing, as described above), integrating governing equations over each control volume to build algebraic equations for each variable, and linearizing these equations to yield updated results throughout the domain.

The “SIMPLE” Pressure-Velocity Coupling Solution Method in FLUENT, which was employed in solutions for this study, uses a segregated algorithm for solving momentum and continuity. Figure 29 shows a schematic of how pressure and velocity are calculated using the segregated algorithm (ANSYS, Inc., 2009).

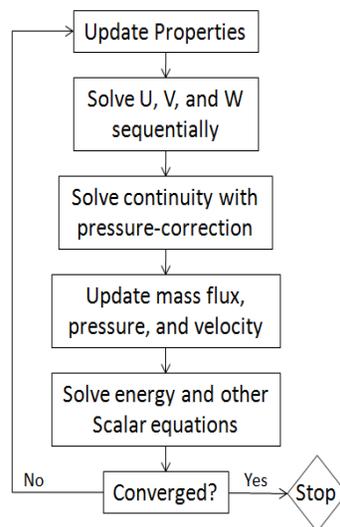


Figure 29: Pressure-based solver used in FLUENT.

FLUENT solves the continuity equation, to a convergence level of 1e-4, as shown in Equation 15.

$$\frac{\partial \rho}{\partial t} + \nabla \cdot (\rho \vec{v}) = S_m \quad \text{Equation 15}$$

where S_m , the mass added via liquid changing phase, will be zero for the simulations performed in this study. For both the pressure-based (used in this paper) and density-based solvers employed by FLUENT, the velocity field is solved directly by the momentum equation.

Pressure fields, on the other hand, are found by manipulating the continuity equations and momentum equations to find a suitable solution. Pressure can be solved directly, or by means of a pressure correction equation.

Conservation of momentum is also solved, to a convergence level of 1e-6, as shown in Equation 16.

$$\frac{\partial}{\partial t} (\rho \vec{v}) + \nabla \cdot (\rho \vec{v} \vec{v}) = -\nabla p + \nabla \cdot (\vec{\tau}) + \rho \vec{g} + \vec{F} \quad \text{Equation 16}$$

where $\vec{\tau}$ is the stress tensor, $\rho \vec{g}$ is the gravitational force, and \vec{F} is body forces. The energy equation is solved using the velocity and pressure profiles described previously, along with several parameters required to be input. Equation 17 shows the process by which the energy equation is solved in FLUENT.

$$\frac{\partial}{\partial t} (\rho E) + \nabla \cdot (\vec{v}(\rho E + p)) = \nabla \cdot (k_{eff} \nabla T) + S_h \quad \text{Equation 17}$$

where k_{eff} is the effective thermal conductivity, S_h is volumetric heat source term, E is mass-averaged energy, and T is mass-averaged temperature.

Results

CFDPost is used to view and analyze the results from FLUENT found in the previous steps. In addition to the powerful tools it contains for displaying data and distributions, CFDPost has the ability to create functions and expressions for more in-depth calculations. The program was used to create output functions for later use.

Automating the calculation of such important parameters simplifies the design process and allows for those values to be used later in the process during Goal-Driven Optimization to drive an optimal design.

Contours were viewed to confirm proper physics were being captured in FLUENT simulations. Figure 30 shows the temperature profile at the center of a row of hot fluid channels with a uniform inlet temperature of 977K and outlet temperatures are significantly lower.

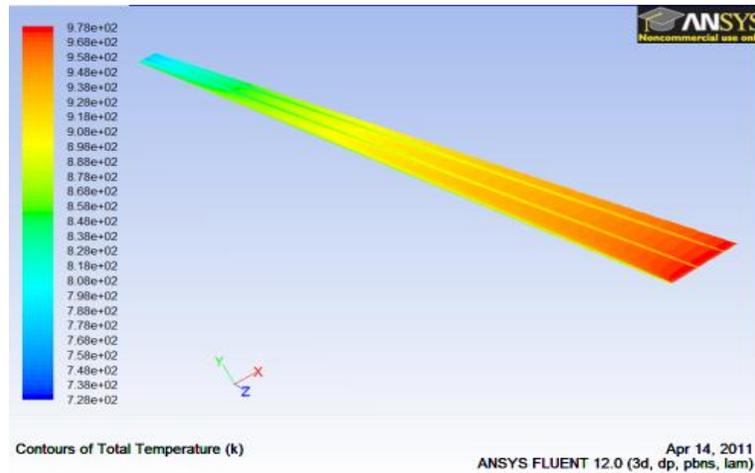


Figure 30: Temperature contour of a hot channel produced from FLUENT.

As expected, temperature gradually changes from the prescribed input value to its final value at the channel outlet. Additional cooling is seen at solid sections that separate flow channels and act as fins to assist in cooling. Thermal boundary layer effects can be seen from the temperature profile as well.

Figure 31 shows the temperature profile developing along the length of a cold channel, where the uniform inlet temperature is shown at 727 K. The same pattern is seen in the cold channel as was seen in the hot channel in Figure 30.

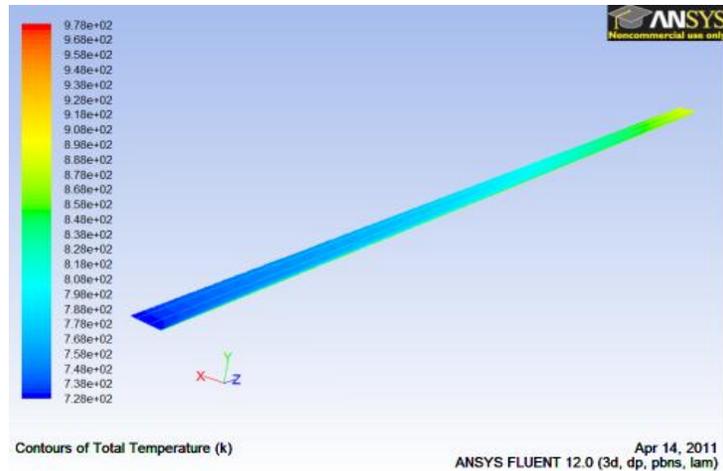


Figure 31: Temperature contour of cold channel produced by FLUENT.

The velocity profiles for each fluid channel are expected to approach zero at their boundaries and their maximum at the center of each channel. Figure 32 shows the velocity profiles found in the full-scale simulation, which performed as expected.

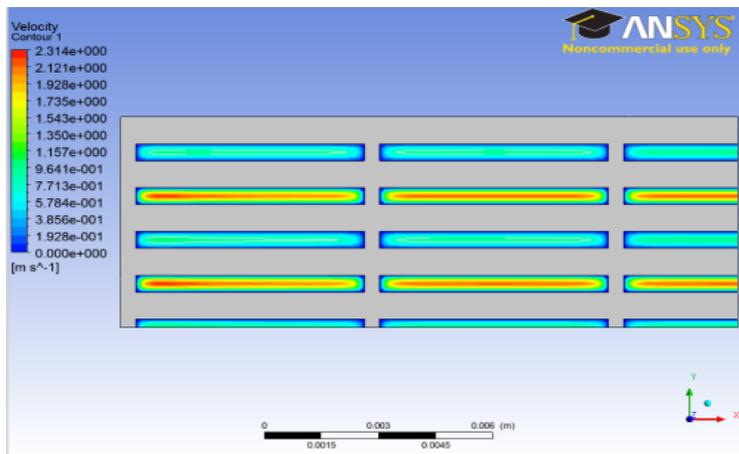


Figure 32: Velocity profile found from FLUENT.

Figure 33 is a temperature profile of the full-scale simulation at half of the total channel length ($Z=0.375$ m). The outer channels are shown to be heated by the outer walls, which are held at 866 K, while the inner channels are transferring heat more efficiently amongst each other.

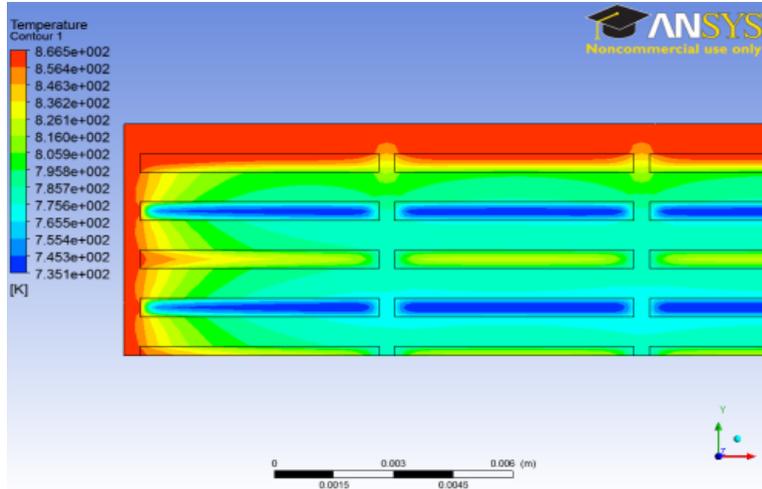


Figure 33: Temperature profile at half channel length shown in CFD Post.

The temperature profile shown in Figure 33 verifies the effect the outer channels felt from the outer wall’s constant temperature boundary condition. With temperatures showing lesser effects from transferring heat with the outer boundary seen in inner channels, the performance of those channels is analyzed more heavily in attempts to bring the simulation to the necessary scale for a nuclear power plant.

Additional contours are provided in Appendix G.

Parameter Set

Several parameters were varied in studies to arrive at an optimal solution. The input parameters are shown and become available for manual or automated manipulation in the Parameter Set. This set of parameters, along with the output parameters produced by each module, is connected to each function in ANSYS Workbench as shown in Figure 21.

Geometric inputs from ANSYS DesignModeler are combined with input/output conditions in FLUENT and problem specifications to create the entire list of input parameters in this study as shown in Table 14.

Table 14: Inlet conditions provided through FLUENT for simulations.

| | |
|------------------------|--|
| Geometric Inputs | H, W, HH, HW, L, SH, SW |
| FLUENT Inputs | $V_{hot, in}$, $V_{cold, in}$, $T_{outside}$, $T_{hot, in}$, $T_{cold, in}$, $P_{hot, out}$, $P_{cold, out}$ |
| Problem Specifications | Q |

As input values are parameterized and output parameters are created in FLUENT and CFD Post, each value is introduced into the “Parameter Set,” which is available for all functions in ANSYS Workbench. In the Parameter Set, “Design Points” are created and solved using the previous steps in Workbench.

In addition to being able to input a variety of parameters for different geometries and operating conditions, derived output parameters can be created to compare different heat exchanger designs. Several factors can be used to determine the performance of a heat exchanger, and they will be explained in detail here.

The parameters contained in the Parameter Set are broken down into several categories: physical properties, FLUENT measurements, intermediate parameters, and output parameters. Physical properties are mass-averaged fluid characteristics found in each fluid channel, then averaged for each fluid. The FLUENT measurements are surface-averaged parameters found at fluid inlets, outlets, and shadow zones. These values were later combined with other measurements and properties to create intermediate parameters, which later helped to create output parameters.

The output parameters found in the Parameter Set are displayed in Table 15.

Table 15: Intermediate and outputs parameters found from FLUENT simulations.

| | |
|-------------------------|--|
| Physical Properties | $\rho_{\text{hot, ave}}, \rho_{\text{cold, ave}}, \mu_{\text{hot, ave}}, k_{\text{ave}}, \mu_{\text{cold, ave}}, C_{p, \text{hot, ave}}, C_{p, \text{cold, ave}}$ |
| FLUENT Measurements | $\bar{h}_{\text{hot, ave}}, \bar{h}_{\text{cold, ave}}, \bar{V}_{\text{cold}}, \bar{V}_{\text{hot}}, \bar{P}_{\text{cold, in}}, \bar{P}_{\text{hot, in}}, \bar{T}_{\text{cold, out}}, \bar{T}_{\text{hot, out}}$ |
| Intermediate Parameters | $\bar{\Delta T}_{\text{hot}}, \bar{\Delta T}_{\text{cold}}, Re_{D_h \text{ hot}}, Re_{D_h \text{ cold}}, D_h, \frac{SA}{VOL}, \frac{SA}{MAT}, U, \Delta T_{LMTD}, \dot{m}_{\text{hot, ave}}, \dot{m}_{\text{cold, ave}}$ |
| Output Parameters | $VOL_{\text{required}}, MAT_{\text{required}}, \varepsilon, VOL_{\text{fuel salt}}, \bar{\Delta P}_{\text{hot}}, \bar{\Delta P}_{\text{cold}}$ |

In order to evenly compare different compact heat exchanger designs created in this study, a constant mass flow rate was maintained for all designs. The mass flow rate can be divided into different channel shapes and geometries, but the total value will remain constant when considering different designs for a similar problem. Mass flow rate is calculated as seen in Equation 18 and is manipulated by changing channel dimension and inlet velocities (V_{in}).

$$\dot{m} = \rho A_C V \quad \text{Equation 18}$$

where ρ is fluid density, A_C is the cross-sectional area of the flow, and V is the fluid velocity. The flow rate relates to the amount and speed of heat that needs to be removed from the incoming flow.

Equation 19 shows an important parameter, the hydraulic diameter, which is vital in calculating several hydrodynamic characteristics of a flow pattern. The hydraulic diameter is vital in developing a representative geometric “diameter” for non-circular cross-sectional areas.

$$D_h = \frac{4A_C}{P} \quad \text{Equation 19}$$

where A_C is the cross-sectional area of the flow and P is the wetted perimeter of the flow channel. The hydraulic diameter of flow in a circular cross-section is simply equal to the diameter of the channel.

Hydraulic diameter is one factor in determining the Reynolds number, Re_{D_h} , a dimensionless factor that describes the turbulence of flow. Equation 20 describes the Reynolds number, which was closely monitored throughout testing.

$$Re_{D_h} = \frac{\rho V D_h}{\mu} \quad \text{Equation 20}$$

where ρ is the fluid density, V is the fluid velocity, D_h is the hydraulic diameter, and μ is the fluid viscosity. Reynolds number was monitored for each design to maintain a value below the laminar-turbulent transition value. The exact laminar-turbulent transition value is not well-known, and is often a basis for disagreement among those who have investigated it, but a simple linear interpolation between two lesser-disputed data points was offered as a means for determining the value over a large range of rectangular channel cross-sections (Kandlikar, Garimella, Li, Colin, & King, 2006).

The transitional Reynolds number, Re_t , for a square channel (aspect ratio $\alpha_c = 1$) has been found to be 2200. For parallel plates ($\alpha_c = 0$), the transitional Reynolds number has been shown to be 2500 (Kandlikar, Garimella, Li, Colin, & King, 2006). For any value between these two points, a simple linear interpolation was completed. A full explanation of the laminar-turbulent transition is provided in several texts (Kandlikar, Garimella, Li, Colin, & King, 2006). The physics behind transitional flow is still not fully known at this point, and is not expected to play a vital role in

this study. A brief discussion, along with the correlation used for rectangular channels used in this study, will be presented in Appendix F.

A number of other output parameters can be calculated by combining outputs from FLUENT to create meaningful comparative measures. One such measure is heat exchanger effectiveness, ϵ , which describes how completely heat is transferred from one fluid to the other (Kays & London, 1984), is shown in Equation 21.

$$\epsilon = \frac{(\dot{m}C_p)_{\text{hot}}(T_{\text{in}}-T_{\text{out}})_{\text{hot}}}{(\dot{m}C_p)_{\text{min}}(T_{\text{hot,in}}-T_{\text{cold,in}})} = \frac{(\dot{m}C_p)_{\text{cold}}(T_{\text{in}}-T_{\text{out}})_{\text{cold}}}{(\dot{m}C_p)_{\text{min}}(T_{\text{hot,in}}-T_{\text{cold,in}})} \quad \text{Equation 21}$$

where \dot{m} is the mass flow rate of the fluid, C_p is the specific heat of the fluid, and T is the temperature of the fluid. Compact heat exchangers can achieve effectiveness of up to 0.99, which stems from the very close approach temperatures achieved by such designs (Wang, Sunden, & Manglik, 2007).

Preliminary computations in this study compare heat exchanger channels of both circular and slotted designs, so the following two computations will introduce geometric calculations for each cross-section.

Another key measure in this study is compactness, or the surface-area-to-volume ratio for a given arrangement. Compactness offers perspective on the ability of a given design to contain a large amount of usable contact surface area for heat transfer in a given three-dimensional volume. The importance of compactness, which is given for a slotted design in Equation 22 and a circular design in Equation 23 will be explained later in this report.

$$\left(\frac{\text{SA}}{\text{VOL}}\right)_{\text{slot}} = \frac{N_W N_H (2W + 2H)}{[N_W W + SW(N_W + 1)][N_H H + SH(N_H + 1)]} \quad \text{Equation 22}$$

where W is width of a slotted channel, H is the height of a slotted channel, SW is the spacing width between channels, SH is the spacing height between channels, N_W is the number of channels wide, and N_H is the number of channels in height.

$$\left(\frac{\text{SA}}{\text{VOL}}\right)_{\text{circles}} = \frac{N_W N_H \pi D}{[N_W D + SW(N_W + 1)][N_H D + SH(N_H + 1)]} \quad \text{Equation 23}$$

where D is the diameter of a circular channel, SW is the spacing width between channels, SH is the spacing height between channels, N_W is the number of channels wide, and N_H is the number of channels in height.

Another vital measure of the performance of a heat exchanger is the ratio of surface-area-to-material-volume. When using materials that are as expensive as Hastelloy N, limiting the amount of material used is a vital consideration in the final design. The relative importance of maximizing SA/MAT will be explained in detail later in this report. Equation 24 shows the manner in which SA/MAT can be solved for slotted designs, respectively.

$$\left(\frac{SA}{MAT}\right)_{\text{slot}} = \frac{N_W N_H (2W + 2H)L}{VOL - (N_W N_H H W L)} \quad \text{Equation 24}$$

where VOL is the volume of the given arrangement, L is the channel length, H is the slotted channel height, W is the slotted channel width, N_W is the number of channels wide, and N_H is the number of channels in height. Equation 25

$$\left(\frac{SA}{MAT}\right)_{\text{circles}} = \frac{N_W N_H \pi D L}{VOL - \left(\frac{\pi D^2}{4} N_W N_H L\right)} \quad \text{Equation 25}$$

where VOL is the volume of the given arrangement, L is the channel length, D is the diameter of a circular channel, N_W is the number of channels wide, and N_H is the number of channels in height. The total amount of heat removed from a heat exchanger can be found by using Equation 26.

$$Q = UA_{HT} \Delta T_{LMTD} \quad \text{Equation 26}$$

where A is the surface area on which heat transfer can occur, U is the overall heat transfer coefficient, and ΔT_{LMTD} is the log-mean temperature difference that is defined in Equation 27 (Kays & London, 1984).

$$\Delta T_{LMTD} = \frac{\Delta T_a - \Delta T_b}{\ln\left(\frac{\Delta T_a}{\Delta T_b}\right)} \quad \text{Equation 27}$$

where T is the temperature, “a” denotes one fluid stream, and “b” denotes the other fluid stream. Log-mean temperature difference is always positive, and is a common tool for describing the thermal inertia existing between flows of different temperatures. As seen in Equation 26, ΔT_{LMTD} is joined by the area between flows, A, and overall heat transfer coefficient, U.

The overall heat transfer coefficient, U, when arranged in such a way that area on either side is exactly the same, can be calculated as seen in Equation 28.

$$\frac{1}{U} = \frac{1}{h_h} + \frac{1}{h_c} + \frac{t}{k} \quad \text{Equation 28}$$

where t is the wall thickness, k is the thermal conductivity of the wall solid, and \bar{h} is the average convective heat transfer for each fluid (where “h” denotes the hot fluid and “c” denotes the cold fluid). Depending on the number of channels used in each simulation, the average convective heat transfer coefficient used in Equation 29.

$$\bar{h}_i = \frac{\sum h_i}{n_i} \quad \text{Equation 29}$$

where h_i is the heat transfer coefficient on the outer surface of each channel and n_i is the number of channels containing fluid “i”.

With a given heat load, Q, and known heat transfer performance, U and ΔT_{LMTD} , a prescribed heat transfer area is then known to be able to size the heat exchanger. Equations 30 and 31 are final outputs from the FLUENT runs that can be optimized, by means of minimizing their output quantities, over an array of design points. The goal of this study is to minimize the amount of material and volume required to house a heat exchanger that can handle the requisite load. Equations 31 and 32 are the ultimate measurement tools for these goals.

$$VOL_{\text{required}} = \frac{A_{HT}}{\frac{SA}{VOL}} \quad \text{Equation 30}$$

$$MAT_{\text{required}} = \frac{A_{HT}}{\frac{SA}{MAT}} \quad \text{Equation 31}$$

where A is the surface area required to remove the heat load with the given heat transfer characteristics, SA/VOL is the compactness for the given setup, and SA/MAT is the corresponding material efficiency. These results can then be compared to approximations made from ORNL design paperwork to compare to that of their shell-and-tube design.

Another vital metric for comparing heat exchanger designs is the volume of the fuel salt required to transfer the requisite amount of heat. As found in Equation 32 shows the process for determining the fuel salt volume, which is found by calculating the volume of one channel and multiplying by the number of fuel channels.

$$\text{VOL}_{\text{fuelsalt}} = \frac{N_W N_H}{2} H W L \quad \text{Equation 32}$$

where H is the slotted channel height, W is the slotted channel width, L is the channel length, N_W is the number of channels in width, and N_H is the number of channels in height. The fuel salt volume directly affects the fissile inventory in a Molten Salt Reactor, so reducing the fuel salt volume can lead to substantial reductions in cost for each plant startup. Transferring heat more efficiently and effectively through a compact heat exchanger can provide significantly lower fuel salt volumes, which will be shown later in this report.

The results found from these equations were automatically updated in the Parameter Set and were later used in conjunction with Goal Driven Optimization (GDO) to find a final design for this project.

Optimization

ANSYS Workbench allows for Goal-Driven Optimization that can assess several goals and objectives in an automated manner. The steps in this process include: Design of Experiments and Goal-Driven Optimization. They will be described in this section.

Design of Experiments (DOE)

Design of Experiments (DOE) is a function of ANSYS Workbench that allows for automated sampling of input parameters over their entire possible range. Instead of manually altering each problem input and combining it with every possible combination of each other input parameter, DOE automatically completes this task and solves each Design Point in series.

Several options are available for sampling the entire possible range of input values, with options built into DOE to most efficiently sample the space. The “Central Composite Design” method

with Central Composite Design was used to optimally fill the sample space (ANSYS, Inc., 2009).

DOE creates a series of data points that cover the entire possible range of each input parameter, which allows for a thorough understanding of the sample space at hand. With the wealth of knowledge from the data points created and solved in DOE, a Response Surface can be built to characterize the impact of each input parameter on each output parameter.

Optimization

A brief optimization was performed for several channel parameters to reach full designs that met the boundary conditions prescribed in ORNL’s design. Future studies can perform more in-depth optimization by using the Goal-Driven Optimization (GDO) module within ANSYS Workbench, but the present study serves to offer possible design points that may be acceptable for a complete design.

Several channel geometries and corresponding boundary conditions were altered in order to reach the necessary target output values and parameters. The relative importance of each target or objective can be defined independently (ANSYS, Inc., 2009). Table 16 shows the GDO goals, limits, and priorities used for this study.

Table 16: Input and output parameters found in this study.

| | Parameter | Optimization Goal - Priority Level |
|----------------------------|-------------------|---|
| Inputs | H | $0.4 < H < 5\text{mm}$ |
| | W | $4 < W < 50\text{mm}$ |
| | SH | $0.8 < SH < 2\text{mm}$ |
| | SW | $0.4 < SW < 2\text{mm}$ |
| | P_{out} | 0 |
| | V_{in} | Maintain $Re < 2000$ |
| | Q | 528.5MW |
| Intermediate Values | SA/VOL | -- |
| | SA/MAT | -- |
| | U | -- |
| | ΔT_{LMTD} | Target – 255K |
| | A | |
| Outputs | $VOL_{required}$ | Minimize – High Priority |
| | $MAT_{required}$ | Minimize – Medium Priority |
| | $VOL_{fuelsalt}$ | Minimize – High Priority |
| | ϵ | Maximize – Medium Priority |
| | ΔP | ~ ORNL value – Low Priority |

The results of the GDO module will be presented in the subsequent section of this report as results are reached for a full-scale design.

Results and Discussion

Three different studies were performed: model validation, small-scale comparison of channel geometry, and full-scale comparison. The model validation step consists of comparing the results from FLUENT to known correlations produced from other studies on rectangular channels. The small-scale comparison looks at the performance of a circular cross-section channel and slotted cross-section channels that were modeled in the first study. The third study seeks to build a model that can be extrapolated to a full-scale design that can evenly be compared with ORNL's shell-and-tube PHX.

The results for both studies are shown below.

Model Validation Theory

The physics involved in heat transfer and fluid flow are relatively well known for rectangular channels, so a preliminary study was launched to validate the FLUENT code and process used in this study for such a shape. After creating a single rectangular channel, several more rectangular geometries could then be compared to correlations that have been developed for varying channel aspect ratios. From this point, a simple change of geometry and boundary conditions will allow for full-scale results to be simulated and compared to ORNL's design to test the validity of the slotted concept.

The hydrodynamic entry length, L_H , must first be calculated to determine which correlation, developing or fully-developed, should be used for calculations. Velocity profiles develop along any surface as shown in Figure 34, where flow velocities are 0 at the wall (assuming no-slip along the wall) and approach their nominal value once they reach the boundary layer thickness.

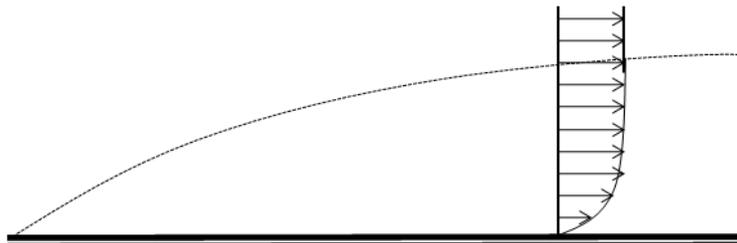


Figure 34: Boundary layer growth over a flat plate.

Flow becomes hydrodynamically fully developed in channel flow when the viscous boundary layers growing from each opposing wall meet in the center of the channel. Figure 35 shows two

viscous boundary layers emerging from opposite sides of a channel, meeting at the center, and becoming fully-developed as shown in the figure.

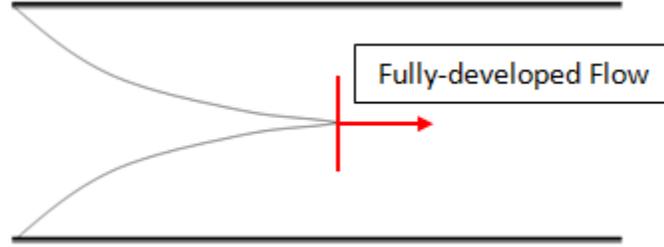


Figure 35: Boundary layer growth in channel flow.

When flow becomes fully developed, velocity profiles will no longer change until some type of perturbation causes the boundary layers to separate and once again begin to form. A common equation for determining the hydrodynamic entry length is found in Equation 33 (Kandlikar, Garimella, Li, Colin, & King, 2006).

$$\frac{L_H}{D_H} = 0.05 \text{Re}_{D_h} \quad \text{Equation 33}$$

For a hydraulic diameter of 1mm and Reynolds number of 1000, as they are expected to be within the typical range of values in this study, L_H would be only 50 mm. With values in this range, it would be reasonable to assume that a 750 mm channel length would be fully developed through the entirety of its length.

The property of importance in validation of the hydrodynamic testing in this study is pressure drop, which can be found easily in CFD simulations. Equation 34 shows the four effects that combine to form pressure drops in typical heat exchangers (Wang, Sunden, & Manglik, 2007).

$$\Delta P_{\text{total}} = \Delta P_f + \Delta P_g + \Delta P_a + \sum \Delta P_{Ni} \approx \Delta P_f + \Delta P_g \quad \text{Equation 34}$$

where ΔP_f is the pressure drop due to friction, ΔP_g is the pressure drop or head due to gravity, ΔP_a is the pressure drop related to flow acceleration within the flow channels, and $\sum \Delta P_{Ni}$ is the sum of pressure drops caused by inlet and outlets. For the models produced and studied in this research, flow acceleration and inlet/outlet conditions are neglected in straight channel sections.

For the preliminary study, gravity has been omitted as a means to isolate the contribution from frictional effects, which can be readily compared to Poiseuille correlations.

Two terms combine to determine the overall pressure drop incurred in a heat exchanger channel. Equation 35 shows the contribution from gravity, which can act to increase pressure drop (when positive) or provide a static head (when negative) (Wang, Sunden, & Manglik, 2007).

$$\Delta P_g = \pm \rho g L \quad \text{Equation 35}$$

The process for determining the frictional contribution to pressure drop is less direct, and requires finding the friction factor, f , which usually involves experimental results or correlations from previous experiments. Equation 36 shows the process by which results for frictional pressure drop, which will dominate the contributions for the purposes of this study, are found (Wang, Sunden, & Manglik, 2007).

$$\Delta P_f = \frac{2fG^2}{\rho D_h} L \quad \text{Equation 36}$$

where G is the mass flux, f is the friction factor, and L is the channel length. In order to simplify the multitude of velocity boundary conditions that may apply to different applications, the dimensionless Poiseuille number was determined for fully developed laminar flow and is shown in Equation 37 (Kandlikar, Garimella, Li, Colin, & King, 2006).

$$Po = f Re \quad \text{Equation 37}$$

where f is the friction factor and Re is the Reynolds number of the flow. The Poiseuille number remains constant for channels of each cross-section and provides a valuable source for finding the friction factor required in Equation 36 for different incoming fluid velocities.

Equation 36 can be combined with Equation 37 to relate the pressure drop directly to flow characteristics and parameters, which will be used to show the empirical Poiseuille results later in this study.

$$Po = \frac{\Delta P_f D_h^2}{2\mu V L} \quad \text{Equation 38}$$

More in-depth studies have been completed on rectangular channels of varying aspect ratio (height-to-width or vice versa), α_c (Kandlikar, Garimella, Li, Colin, & King, 2006). Equation 39 shows empirical results for the Poiseuille number in fully developed flow in rectangular channels of varying aspect ratio, α_c .

$$Po = 24[1 - 1.3553\alpha_c + 1.9467\alpha_c^2 - 1.7012\alpha_c^3 + 0.9564\alpha_c^4 - 0.2537\alpha_c^5] \quad \text{Equation 39}$$

More exact results can be determined when removing the fully developed assumption throughout the pipe (where the Poiseuille correlation can then be used) and solving pressure drop separately in two distinct regions: the developing region and the fully developed region. The apparent friction factor, f_{app} , can be used in developing laminar flow as shown in Equation 40 (Kandlikar, Garimella, Li, Colin, & King, 2006).

$$\Delta P_{f_{app}} = \frac{2f_{app}\rho u_m^2 x}{D_h} \quad \text{Equation 40}$$

where x is the distance from the inlet to the points of interest, usually where flow becomes fully developed, and u_m is the mean fluid velocity. The apparent friction factor acts as an average friction factor over the developing range, and can then be added to the fully developed pressure drop as it occurs over the remaining channel length. A measure of the difference incurred from using the apparent friction factor in place of the fully-developed results is captured in the incremental pressure defect, $K(x)$ (Kandlikar, Garimella, Li, Colin, & King, 2006). Equation 41 shows the process for finding $K(x)$, which can be used later to show the pressure drop across both the developing and fully-developed regions.

$$K(x) = (f_{app} - f) \frac{4x}{D_h} \quad \text{Equation 41}$$

The results from Equation 41 can be combined to segregate the pressure drop results from the developing region (using the incremental pressure defect) and the fully-developed region (using the Poiseuille number) to form Equation 42, which displays the full results for pressure drop.

$$\Delta P_{total} = \frac{2(fRe)\mu u_m x}{D_h^2} + K(x) \frac{\rho u_m^2}{2} \quad \text{Equation 42}$$

The friction factor is heavily dependent on empirical results, and robust correlations have been developed for channels of different cross-section and boundary condition to cover a wide range of applications. Figure 36 shows a known range of correlations for Po as a function of channel aspect ratio (shorter channel height/longer channel width) (Kays & London, 1984) assuming fully developed flow in the rectangular channel.

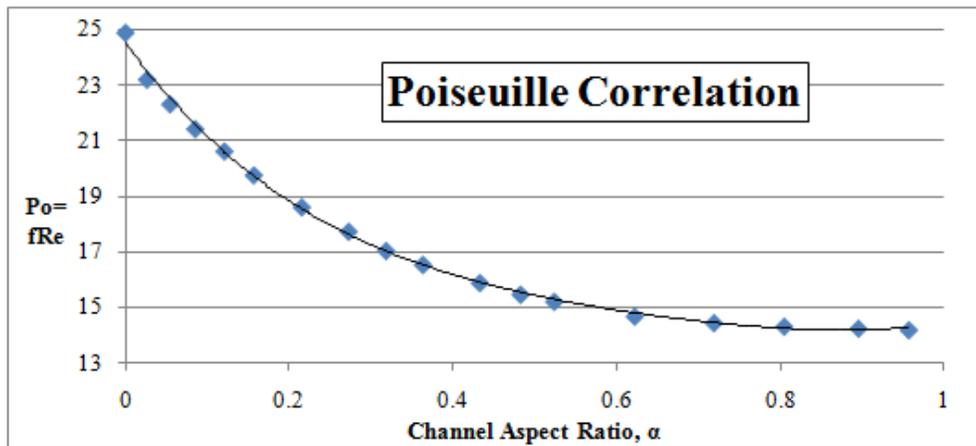


Figure 36: Poiseuille correlation for varying rectangular channel aspect ratio (Kays & London, 1984).

Hydrodynamic effects can be equally compared to the results seen in Figure 36 for fully developed flow, but the heat transfer (and corresponding thermal development region) cannot use the same assumption.

Thermal entry length calculations are performed in a similar manner to that of the hydrodynamic development, which was explained previously. Analogous to Figure 35 above for hydrodynamic flow development, channel flow becomes thermally developed once its thermal boundary layers meet in the center of the pipe. Figure 37 shows both the hydrodynamic and thermal boundary layers of a channel flow for a fluid with a Prandtl number, a measure of the diffusivity of momentum relative to heat, greater than 1 (a Prandtl number higher than 1 means that momentum is diffused more easily than temperature, which leads to thermal boundary layer becoming fully developed *after* the thermal boundary layer).

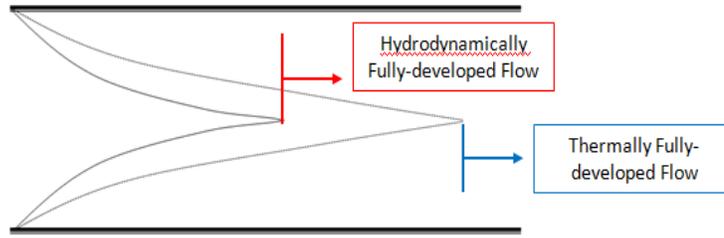


Figure 37: Thermally-developing channel flow for high-Prandtl number fluid.

Equation 43 shows the thermal entry length for fully developed laminar fluid flow in a rectangular channel (Kandlikar, Garimella, Li, Colin, & King, 2006).

$$\frac{L_T}{D_H} = CRePr \quad \text{Equation 43}$$

where C is 0.05 for circular channels and 0.1 for rectangular channels, Re is the Reynolds number of the flow and Pr is the Prandtl number of the fluid (Kandlikar, Garimella, Li, Colin, & King, 2006).

The preliminary study will be tested with mass-averaged flow velocities of approximately 1m/s and hydraulic diameters of less than 1mm over the 0.5 m length scale. The small-scale model used in the validation study will become fully developed at a distance of approximately 500 mm, which is almost identical to the channel length being tested in the preliminary study. Full-scale models will have even longer thermal development regions, so the same thermally developing correlations will hold true in those studies as well.

Whereas hydrodynamic validation is performed by finding the dimensionless Poiseuille number, which can be related directly to pressure drop, thermal validation is performed using the Nusselt number. The Nusselt number compares the contributions of convective and conductive heat transfer and is known to vary with changing channel cross-section and boundary conditions selection, while remaining constant in fully developed flow. The Nusselt number is found according to Equation 44.

$$Nu_L = \frac{hL}{k} \quad \text{Equation 44}$$

where h is the heat transfer coefficient, k is the thermal conductivity, and L is the characteristic length in the direction in which the thermal boundary layer growth of interest occurs.

In this research, thermal boundary layers form along the length axis and the width axis and will reach fully developed flow once they each grow to be half the height of the channel's cross-section. Both the Reynolds number and Nusselt number will be calculated based on the hydraulic diameter in this study.

Difficulties arise in producing accurate Nusselt correlations due to the sensitivity of heat transfer to the boundary conditions used in each problem set up. Correlations have been produced for four-sided heating (heat flux is occurring from the every outer surface of the channel) and three-sided heating (one side is kept adiabatic, while the other three transfer heat) in developing flow situations. Steady-state flow Nusselt correlations can use constant wall temperature, constant heat flux (both axially and circumferentially), or uniform axial heat flux and constant circumferential wall temperature (Kandlikar, Garimella, Li, Colin, & King, 2006).

The boundary conditions for any problem will most likely be a combination of the above-mentioned boundary conditions and adding a different and larger array of channels will add yet another factor to change results. For this reason, a simple study of one channel should be sufficient to validate the process used in this study. The characteristics used in correctly finding those results can then be expanded to a larger geometry, which can find the results for the geometry and boundary condition at hand in this study. The Nusselt correlation found from those results can then be presented for the specifics used in this research.

The process by which the full-scale model's Nusselt number is found must start again at determining the regime in which the thermal boundary layer growth will render the flow.

For channel hydraulic diameters on the order of 1 mm, Reynolds numbers maintained at approximately 1000, and Prandtl numbers typically being 15, the thermal entry length in this study can be found to be 1.5m by using Equation 43 Flow over the 0.75 m length used in this study can then be assumed to be developing for the entirety of the channel length in each design that will be tested. Such an assumption will require the use of correlations for thermally developing flow, which differ from the fully developed flow correlations.

Thermally developing flow will have Nusselt correlations depending on channel aspect ratio, α_c , and non-dimensional thermal length, Equation 45 shows the process by which this length is found.

$$\mathbf{x}^* = \frac{\mathbf{x}/D_h}{\mathbf{RePr}} \quad \text{Equation 45}$$

where x is the distance along the length of the channel and Pr is the Prandtl number of the fluid. With a channel aspect ratio prescribed and characteristics of the flow and fluid known, Nusselt correlations are tabulated as listed in Table 17 (Kandlikar, Garimella, Li, Colin, & King, 2006).

Table 17: Nusselt correlations for varying channel aspect ratio and non-dimensional length (Kandlikar, Garimella, Li, Colin, & King, 2006).

| x^* | $\alpha_c \leq 0.1^*$ | $\alpha_c = 0.25$ | $\alpha_c = 0.333$ | $\alpha_c = 0.5$ | $\alpha_c = 1$ | $\alpha_c \geq 10^{**}$ |
|----------------|-----------------------|-------------------|--------------------|------------------|----------------|-------------------------|
| 0.0001 | 31.4 | 26.7 | 27.0 | 23.7 | 25.2 | 31.6 |
| 0.0025 | 11.9 | 10.4 | 9.9 | 9.2 | 8.9 | 11.2 |
| 0.005 | 10 | 8.44 | 8.02 | 7.46 | 7.1 | 9.0 |
| 0.00556 | 9.8 | 8.18 | 7.76 | 7.23 | 6.86 | 8.8 |
| 0.00625 | 9.5 | 7.92 | 7.5 | 6.96 | 6.6 | 8.5 |
| 0.00714 | 9.3 | 7.63 | 7.22 | 6.68 | 6.32 | 8.2 |
| 0.00833 | 9.1 | 7.32 | 6.92 | 6.37 | 6.02 | 7.9 |
| 0.01 | 8.8 | 7 | 6.57 | 6.05 | 5.69 | 7.49 |
| 0.0125 | 8.6 | 6.63 | 6.21 | 5.7 | 5.33 | 7.2 |
| 0.0167 | 8.5 | 6.26 | 5.82 | 5.28 | 4.91 | 6.7 |
| 0.025 | 8.4 | 5.87 | 5.39 | 4.84 | 4.45 | 6.2 |
| 0.033 | 8.3 | 5.77 | 5.17 | 4.61 | 4.18 | 5.9 |
| 0.05 | 8.25 | 5.62 | 5.00 | 4.38 | 3.91 | 5.55 |
| 0.1 | 8.24 | 5.45 | 4.85 | 4.22 | 3.71 | 5.4 |
| 1 | 8.23 | 5.35 | 4.77 | 4.11 | 3.6 | 5.38 |

* - simplified to parallel plates, both sides heated ** - simplified to parallel plates, one side heated

Results from this table can be linearly interpolated to arrive at solutions for x^* values at intermediate channel aspect ratios. This empirical method of creating a correlation for this wide variety of channel geometries and flow characteristics is seen as the most viable method in thermally developing flows. Whereas this entire chart was developed for flow in three-sided or four-sided heating environments, a similar chart will be developed for the intermediate situation found in the constant outer wall temperature boundary conditions used in this study.

The results provided from the micro-scale studies previously produced are shown in Figure 38 as a function of inverse dimensionless length. It is shown that Nusselt number decreases steadily with an increased dimensionless length, x^* , until the flow becomes thermally fully-developed ($x^*=1$) as shown in Figure 37, where interpolated results from Table 17 for $\alpha_c = 0.317$ are shown.

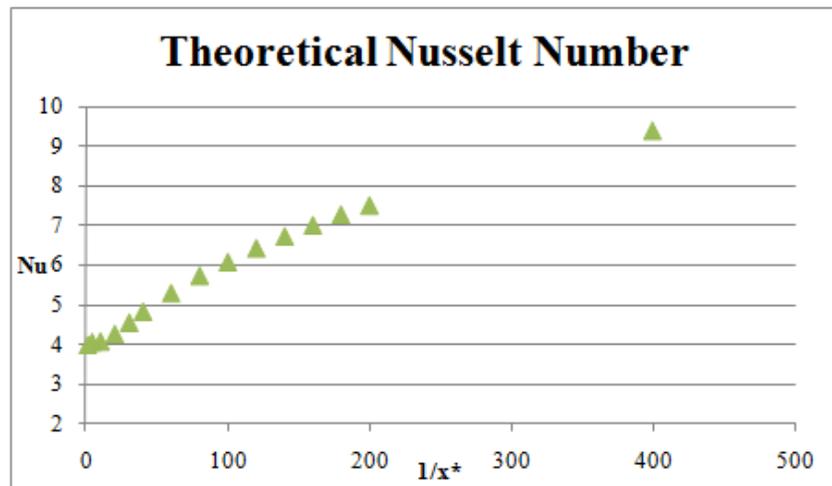


Figure 38: Theoretical Nusselt number as a function of the inverse of dimensionless length (Kandlikar, Garimella, Li, Colin, & King, 2006).

It is also useful to compare results from a more concisely presented study to validate the data found in this study. The information found in Table 17 and Figure 38 will be useful in comparing final results as a check for general agreement, but more concisely explained studies are available for small-scale validation.

A rectangular micro-scale study performed by Bhanja to validate Nusselt correlations was repeated and altered slightly to attain validation for the process and mesh used in this study (Bhanja, 2009). Bhanja's study provides a concise process for creating the geometry and boundary conditions for a constant heat flux problem set up that was solved in the study.

A microchannel of 57 μm in height and 180 μm in width was tested over a 10 mm length

From this study, results were first matched to a study that modeled only one channel with constant heat flux on the channel periphery and had its set up readily available (Bhanja, 2009).

Varying the channel inlet flow rate (and velocity) for a constant-size rectangular channel will produce a correlation relating Nusselt number and Reynolds number. Reproducing these results over a range of velocities and heat flux values will assure that the correlation is correct.

Figure 39 shows the Nusselt correlation as a function of Reynolds number provided for the constant heat flux case modeled by Bhanja, which can be compared for a similar problem set up prepared in this study.

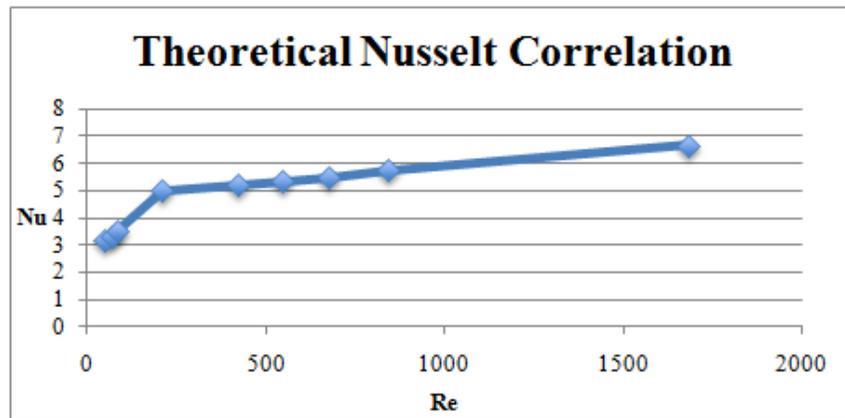


Figure 39

The set up was matched to attempt to produce similar results to validate the methods performed in this study. With a slightly different mesh and set up, simulations must be performed to assure that the physics within the model at hand are accurately captured. The results of this step, as well as the hydrodynamic validation, are shown below.

Model Validation Results

For the purposes of validation needed for the CFD model created and utilized in this study, the results for slotted channels were compared to the correlations presented previously in this section. In addition to providing assurance that they physics have been appropriately captured, the mesh parameters and solution methods used can be confirmed as satisfactory for extension to larger geometries.

A wide range of channel dimensions and inlet velocities were tested to assure agreement with well known correlations. Equation 36 was used to find the Poiseuille number from pressure drops found in FLUENT. Figure 40 shows the results for the Poiseuille number obtained from FLUENT in this study, as well as a known Poiseuille correlation for rectangular channels (Kays & London, 1984).

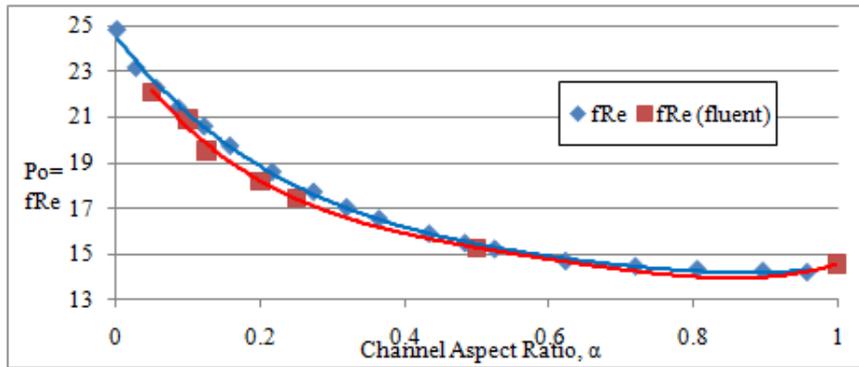


Figure 40: Poiseuille correlation results found in FLUENT validation.

These results show a general agreement in both value and trend with the known values produced in previous studies. Percent error is kept below five percent at all data points, and a polynomial fit was developed that nearly provides an exact match for the correlations produced from the studies performed by Kays and London.

Figure 41 shows the results found in this study and compares them to those found in a previous study of rectangular channels (Bhanja, 2009).

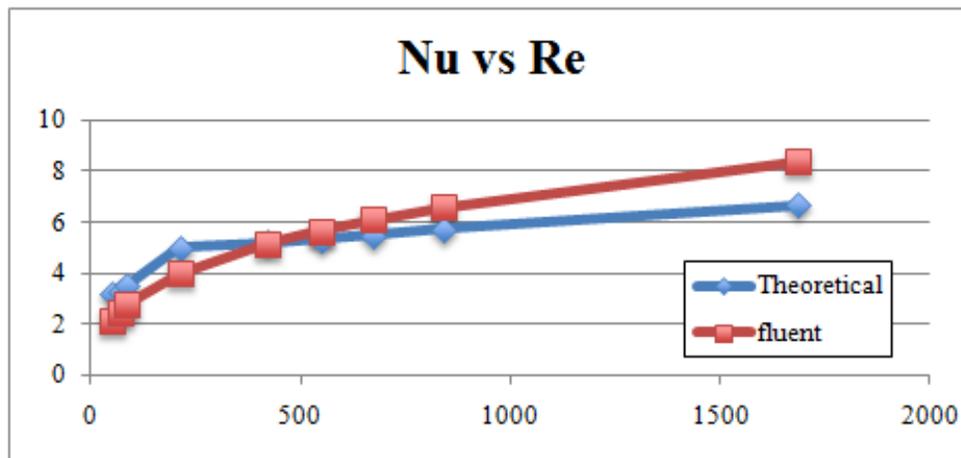


Figure 41: Nusselt correlation for varying Reynolds number in micro-scale rectangular channel (Bhanja, 2009).

With these results, the theory and mesh have shown to provide results roughly equal to those produced by previous studies. The model and set up can then be updated to include a larger

geometry and slightly different boundary conditions to simulate small-scale rectangular minichannels, as well as a relatively large array of full-scale channels.

Comparison of Circular and Slotted Designs

The first step in moving from modern compact heat exchangers to a slotted design is to compare the performance of a slotted heat exchanger to that of the more traditional circular cross-section channels. Due to eddy current inspections being further developed in circular cross-sections than high aspect ratio slotted designs, it may be an advantage to stay with circular channels. All designs were kept with constant total mass flow rates and equal hydraulic diameters in an attempt to produce similar Reynolds number and pressure drop values. Slotted designs of high aspect ratio (W/H) and low aspect ratio were tested.

With a channel diameter of 1 mm determined for the circular channels, the high aspect ratio slotted channel arrangement was brought to channel widths of 11.56 mm and channel heights of 0.5225 mm to arrive at the same hydraulic diameter (1 mm) while keeping a channel aspect ratio in excess of 10 for the slotted design. The low aspect ratio channel arrangement has a channel height of 0.75 mm and a width of 1.5 mm, arriving at the same hydraulic diameter of 1 mm.

Maintaining similar hydraulic diameters for different channel geometries is important due to the need to compare the overall performances designs can offer while providing similar pressure drops. As a slotted channel is increased in channel width (W), thus becoming thinner and more infinite in aspect ratio, SA/VOL and SA/MAT values will become increasingly better. As the geometric properties are improved by thinning and widening the channel, hydraulic diameter increases to limit the velocity of the fluid to maintain laminar flow, adversely affecting performance. Finding the optimal balance between these two effects will be done by Goal Driven Optimization in ANSYS Workbench due to its ability to thoroughly exhaust all available design options.

In order to keep run times reasonable for a preliminary study, only a 200 mm length was modeled to compare the two designs. Relative heat transfer performance between two designs will remain similar over any length of pipe when the fluids and boundary conditions are kept equal, and the ability to model only a portion of the channel length is a significant advantage. These results will not be relevant for full-scale discussion and comparison, but will remain relevant to show that a slotted heat exchanger can compete with more well known circular cross-sections that may be unusable for this application due to fouling and blockage from loose impediments.

Figure 42 show the circular cross-section geometry that was compared to the slotted geometries in Figure 24 on the 200 mm length scale for the preliminary comparison.

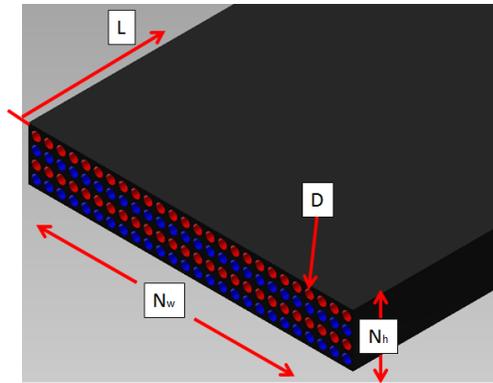


Figure 42: Circular cross-section channels used in small-scale study.

Opposing channel compositions are shown in red (“hot” fluid) and blue (“cold” fluid) in an alternating fashion as discussed previously. The two fluids in this short heat exchanger section, without headers or common plenums at this point, are acting in pure counterflow. A different number of channels were required to handle the mass flow for each of the designs. Dissimilar heat transfer and hydrodynamic performance are expected from the two designs due to the change in geometry and distribution of the fluid flow.

The design inputs and resulting metrics are shown in Table 18 for the circular and two slotted channel designs.

Table 18: Results from small-scale study.

| Parameter | Slotted (High W/H) | Slotted (Low W/H) | Circular | |
|--|-----------------------|----------------------|----------|---------|
| D/H/W (mm) | -- /0.5225/11.56 | --/0.75/1.5 | 1/--/-- | Inputs |
| L (mm) | 200 | 200 | 200 | |
| \dot{m}_{total} (kg/s) | 0.5257 | 0.5257 | 0.5257 | |
| N_H | 4 | 4 | 4 | |
| N_W | 4 | 4 | 16 | |
| Q (kW) | 1000 | 1000 | 1000 | |
| | | | | |
| $\Delta P_{average}$ (kPa) | 403 | 380 | 420 | Outputs |
| U (W/m ² K) | 934 | 1319 | 889 | |
| ΔT_{LMTD} (K) | 55.1 | 47.0 | 40.0 | |
| A_{HT} (m ³) | 19.4 | 15.3 | 28.1 | |
| ϵ | 0.240 | 0.195 | 0.183 | |
| SA/VOL (m ² /m ³) | 1316 | 1286 | 1334 | |
| SA/MAT (m ² /m ³) | 1961 | 3600 | 1647 | |
| $VOL_{required}$ (m ³) | 0.0147 | 0.0119 | 0.0210 | |
| $MAT_{required}$ (m ³) | 0.00990 | 0.00425 | 0.0171 | |
| $VOL_{fuel\ salt}$ (m ³) | 9.7E-06 | 1.8 E-06 | 5.0 E-06 | |

Initial designs used aggressively high aspect ratios, which present more of an improvement in SA/VOL and SA/MAT, but their heat transfer performances are adversely affected by their larger hydraulic diameter. This leads to a decrease in the fluid velocity permitted to maintain laminar flow, thus decreasing the ability to transfer heat efficiently.

Entering into the turbulent flow regime, by means of raising the fluid inlet velocity, is an option to increase the heat transfer from the slotted design. Although channel lengths would be decreased due to the elevated pressure drop resulting from turbulent flow, heat transfer performance would be expected to improve.

The slotted heat exchanger performed better than the circular design according to nearly all metrics used in this study. Advantages were seen in volume and material requirements, although an increase in fuel salt volume was seen in the high aspect ratio design. The results found in this preliminary study are sufficiently promising to move on to a full-scale array of channels, which can then be directly compared to ORNL's full shell-and-tube design.

Full-Scale

The ability to show that an immature, un-optimized slotted design can perform to nearly the level of the well-known shell-and-tube heat exchanger would be a significant accomplishment in showing the long-term capabilities held by these heat exchangers. Significant studies have been conducted on maximizing and inducing turbulence on the shell-side flow of the shell-and-tube heat exchanger, which is a significant advantage that the slotted design does not yet have.

Moving forward with the slotted design, a move to a full-length section of the heat exchanger was modeled and analyzed, with a channel length of at least 0.5 m but not to exceed 1 m. As the length of the section was increased to that of an expected full-scale heat exchanger, the channel dimensions were commensurately changed. A circular channel design would be a viable option for many applications, mostly as a simple compromise between ultra-compact heat exchangers and shell-and-tube heat exchangers, but the slotted design is optimal for this study because of the goal of minimizing material volume. The high cost of Hastelloy N is substantial relative to other major system components, and can be decreased drastically with a slotted compact design.

The first design was optimized to obtain the correct output temperature boundary conditions with channel heights below the current eddy current limit of 1.8 mm. To maintain a high aspect ratio, the channel width was found to perform appropriately at 18 mm. In order to counter-act the much lower heat transfer coefficient of the large channel, the channel length was lowered to 0.5 m.

With in-depth data tables provided by ORNL, the heat transfer quantities can be compared for performance (Bettis, et al., 1967). Detailed sketches in the ORNL paperwork provide enough data to give rough estimates on heat transfer area, exchanger volume, and exchanger material inventory (neglecting headers and the 14 donut baffles, which would further any advantage in favor of the slotted design). Rough calculations used to find the volume and material usage required for the ORNL PHX design are shown in Appendix D.

Table 19 displays the results from the final slotted design and compares them to ORNL's shell-and-tube design for each of the four PHX modules.

Table 19: Results from full-scale study in comparison to ORNL’s design.

| | ORNL | Slotted (5x9) | Slotted (Scaled) |
|------------------------------|-------------|--------------------------|-----------------------------|
| L (m) | 5.0 | 0.75 | 0.75 |
| Q (MW_t) | 528.5 | 0.0585 | 528.5 |
| D/H/W (mm) | 9.525/--/-- | --/1.8/18 | --/1.8/18 |
| | | | |
| U (W/m^2K) | ~2791 | 87 | ~87 |
| ΔT_{LMTD} (K) | 155.6 | 155.8 | ~155.8 |
| A_{HT} (m^2) | ~1,136 | 0.127 | 41,158 |
| SA/VOL (m^{-1}) | ~55 | 764 | 828 |
| SA/MAT (m^{-1}) | ~211 | 2041 | 2563 |
| ε | 0.580 | 0.584 | ~0.584 |
| $VOL_{required}$ (m^3) | ~20.7 | 1.7E-04 | 49.7 |
| $MAT_{required}$ (m^3) | ~5.38 | 6.2E-05 | 16.1 |
| $VOL_{fuel\ salt}$ (m^3) | 2.3 | 1.9E-04 | 16.8 |
| ΔP_{COLD} (kPa) | 255 | 2.02 | 2.02 |
| ΔP_{HOT} (kPa) | 600 | 0.924 | 0.924 |

The results found in Table 19 show that channel heights in the currently maintainable regime will provide output metrics that would not present any improvements. Pressure drop, the sole output metric that is improved in this design, renders the design insufficient due to the relatively low importance of minimizing pressure drop. The 1.8 mm channel height can only provide an overall heat transfer coefficient of roughly $87 W/m^2K$, which proved to be too low to be overcome by material efficiency advantages.

Subsequent designs lowered the possible range of channel heights from 0.4 mm to 1.8 mm, when it was previously proven that an improved design could not be provided above these dimensions. Table 20 shows the results found when the channel height was lowered to 0.5 mm.

Table 20: Results from full-scale study in comparison to ORNL’s design.

| | ORNL | Slotted (5x9) | Slotted (Scaled) |
|------------------------------|-------------|--------------------------|-----------------------------|
| L (m) | 5.0 | 0.75 | 0.75 |
| Q (MW_t) | 528.5 | | 528.5 |
| D/H/W (mm) | 9.525/--/-- | --/0.5/6 | --/0.5/6 |
| | | | |
| U (W/m^2K) | ~2791 | 289 | 348 |
| ΔT_{LMTD} (K) | 155.6 | 155.5 | ~155.5 |
| A_{HT} (m^2) | ~1,136 | 1.01E-04 | 10,290 |
| SA/VOL (m^{-1}) | ~55 | 1,333 | 1,562 |
| SA/MAT (m^{-1}) | ~211 | 1,926 | 2,441 |
| ϵ | 0.580 | 0.582 | ~0.582 |
| $VOL_{required}$ (m^3) | ~20.7 | 7.6E-08 | 6.6 |
| $MAT_{required}$ (m^3) | ~5.38 | 5.3E-08 | 4.2 |
| $VOL_{fuel\ salt}$ (m^3) | 2.3 | 1.8E-05 | 1.2 |
| ΔP_{COLD} (kPa) | 255 | 287 | 287 |
| ΔP_{HOT} (kPa) | 600 | 649 | 649 |

The results shown in Table 20 will provide benefits in nearly every metric measured in this study. Although smaller eddy current probes will be required to be developed for this design, even a design with channel heights of 1.8 mm will require the development of small-scale pancake probes that are not currently available. The size of this design will ultimately be determined by the fouling limit, casting limit of wall thickness, and eddy current probe size. The exact dimension at which these limitations come into effect is not explicitly known at this point and will require experimental testing in the future.

Several of the properties taken from ORNL paperwork (Kasten, Bettis, & Robertson, 1966) are approximated due to the constant evolution of the design of the PHX. Several earlier studies list values that differ from later Progress Reports, with relatively significant changes taking place along the way. For example, the 1967 Progress Report mentions that the thermal conductivity of the molten salts was over-estimated in earlier studies. This was said to have resulted in a 20% reduction in the overall heat transfer coefficient. Updated values were not concisely written in the document, so U was assumed to be 80% of the value found in previous documents (Bettis, et al., 1967).

To approximate a full-scale slotted design, the heat load (Q) was assumed to similar to be that from ORNL’s design. With the log-mean temperature difference from the 5x9 array paired with

an overall heat transfer coefficient scaled to represent its expected value at the necessary number of channels, the heat transfer surface area, A_{HT} , can be found according to Equation 26.

The overall heat transfer coefficient value in the preliminary 5-channel by 9-channel array calculations in Table 20 was found by equally weighing the values of the convection heat transfer coefficient on all channels. Since full-scale arrays of channels (possibly having greater than 1,000 channels in each direction) will be dominated by interior channels and will approach the value of the average of the interior channels, exterior channels can be ignored due to their heightened influence from the outer wall boundary condition.

Figure 43 shows the trend of the heat transfer coefficient approaching its total value as the number of channels approaches 300. With only 45 vertical channels modeled in this study, the values found in FLUENT would be roughly 97% of the true full-scale value (Kays & London, 1984). The heat transfer coefficients in channels adjacent to the outer wall were found to be significantly lower than the inner channels, which would be expected to dominate on a full-scale array. The effect incurred by the phenomena seen in Figure 43 was used in the “scaled” calculations in Table 20.

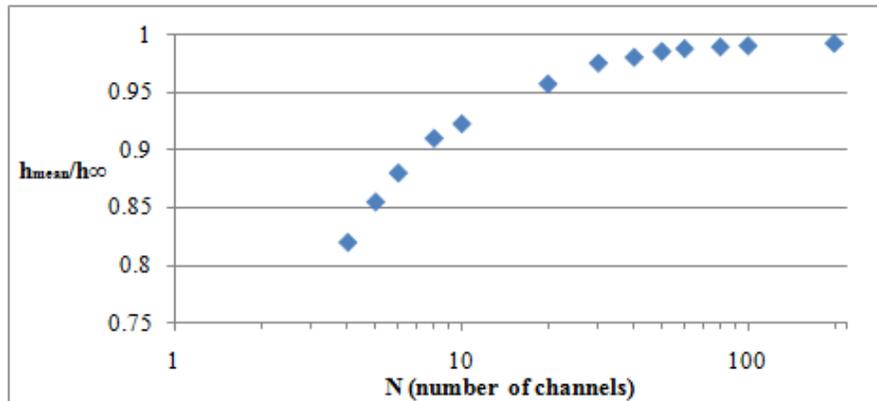


Figure 43: Heat transfer coefficient variation based on number of channels modeled (Kays & London, 1984).

The heat transfer performance of ORNL’s shell-and-tube design was better than the relatively un-optimized slotted design modeled in this study. While it took less than 10% of the material and roughly 3.5% of the volume to accommodate a similar surface area, more surface area was needed for the slotted design because of its lower overall heat transfer coefficient. Pressure drops and effectiveness were maintained to be rather similar for the two designs, with the slotted design having the ability to improve generously with further optimization and freedom in temperature boundary conditions. Results from the 5-channel by 9-channel full-length array showed that, while heat transfer was a detriment, SA/MAT was increased and SA/VOL was

dramatically increased in the slotted design. Fuel salt volume was also brought to a much lower value than that required from the ORNL design.

One detriment to the overall design of the PHX may be the increased pumping power required for the flow rate and pressure drop incurred in the compact heat exchanger. Although the pressure drop was equivalent to ORNL's shell-and-tube design for each fluid, the larger number of channels sees a greater pumping power, as shown in Equation 46.

$$P_{pump} = \frac{\Delta P A_c V}{\eta} \quad \text{Equation 46}$$

where ΔP is the pressure drop for each fluid, A_c is the cross-sectional area of each channel, V is the fluid velocity, and η is the pump efficiency (which was assumed to be 90% for this study). Whereas pumps for the MSBR were designed to be 1400 hp for the secondary coolant and 990 hp for the fuel salt, the slotted heat exchanger design with channel heights of 0.5 mm would require a pump duty increase by a factor of 10 for the secondary coolant (14,748 hp) and 48 for the fuel salt (47,832 hp). These are prohibitively high values that would require the use of larger channels to maintain pump duties on reasonable ranges.

The 1.8 mm channel height, which would also prove to be more robust to fouling and loose impediments, would also lessen the pump requirement that may make it more reasonable for commercial use. Pumping power requirements would be lowered by a factor of almost 16 for the secondary coolant (57.2 hp) and by over 17 for the fuel salt (90.2 hp) for the 1.8 mm channel height design. Future studied can perform more intense optimization to find the value between these two data points that provides a reasonable balance of advantages and drawbacks in that range. The two main data points studied in this thesis can serve as the bounding endpoints for future optimization studies.

Slotted heat exchangers have the ability to be separated into several banks, varied in dimension in each direction, or both. Table 21 shows several such possibilities, where N is the number of banks the slotted heat exchanger is separated into, N_W is the number of channels in the width-direction, N_H is the number of channels in the height-direction, H_{TOTAL} is the resulting height of each bank, and W_{TOTAL} is the resulting width of each bank.

Table 21: Heat exchanger module configuration possible with slotted design.

| N (# banks) | N_W | N_H | H_{TOTAL} (m) | W_{TOTAL} (m) | L (m) | VOL (m^3) |
|------------------------|-------------------------|-------------------------|---------------------------------------|---------------------------------------|------------------|-----------------------------------|
| 1 | 1,027 | 1,027 | 1.3 | 6.6 | 0.75 | 6.64 |
| 4 | 514 | 514 | 0.67 | 3.3 | 0.75 | 6.63 |
| 10 | 325 | 325 | 0.42 | 2.1 | 0.75 | 6.62 |
| 10 | 147 | 721 | 0.94 | 0.94 | 0.75 | 6.63 |
| 16 | 117 | 576 | 0.75 | 0.75 | 0.75 | 6.75 |

The length of the each module is kept constant at the previously-determined optimal value of 0.75 m. The total volume, VOL, is found to change minimally as the heat exchangers are separated into up to 16 different banks. Appendix H presents these options in a scaled, graphical representation that can more succinctly compare the different options available for slotted heat exchanger banks. A sketch of ORNL’s PHX is also provided for comparison. In-depth studies should still be completed regarding the optimization of modular heat exchanger banks. Whereas advantages are seen in separating the exchanger into a large number of banks, corresponding disadvantages are seen in increased piping requirements and header volume.

IV. Conclusions and Recommendations

The results in this study demonstrated the feasibility of an immature, relatively un-optimized slotted heat exchanger to perform at or above the level of ORNL’s shell-and-tube design in all metrics used for comparing performance.

In future iterations of this study, an evaluation of wavy channels with constant channel gaps could be made to enhance heat transfer. If the ability to create a minichannel heat exchanger with heat transfer performance roughly matching that of the shell-and-tube heat exchangers designed by ORNL is shown by a minichannel slotted heat exchanger, a slotted heat exchanger could be shown to be a viable option in commercial nuclear applications. A wavy channel design may have the ability to allow for eddy current inspection techniques to be used along its length, but the wavy nature of the channels may induce turbulence to help heat transfer performance. Another option would be creating very small bars and bumps along the channel surface that will enhance heat transfer. The pressure drop losses and manufacturing challenges are not expected to overcome the increased heat transfer performance.

Further optimization of channel sizes and shapes can be performed to fully optimize the design. ProCast can then be utilized to test the validity of casting Hastelloy N on the scale of 1 mm in wall thickness, which is still an unproven concept. Development of high-aspect-ratio eddy current pancake probes would be another step towards commercial viability.

The ability to contain the PHX within the reactor vessel should be studied in-depth in further studies. The concept was dismissed here due to the complex nature of piping and altered neutron performance in this research, but maintaining the PHX within the reactor vessel but outside of the pool of molten salt, most likely by means of creating a closed portion of the vessel that would protect the heat exchanger and allow for easier maintenance, is preferred. More complete knowledge of the exact plant set-up and reactor dimensions must be known before a study can be launched on this subject.

In the worst possible scenario, a slotted minichannel heat exchanger can present an option for moving to more compact designs that would bridge the time gap until better maintenance and inspection techniques can be developed for ultra-compact designs. The performance available with a slotted heat exchanger is not at the level of other advanced designs, but its ability to more easily be inspected by well-known methods presents an advantage that has been largely ignored up to this point in its evolution. The fabrication and manufacturing techniques proposed for this heat exchanger is expected to offer improvements over advanced techniques necessary for micro-channel heat exchangers and shell-and-tube heat exchangers.

This study has shown an accurate CFD analysis has been carried out to validate the possibility of high aspect ratio rectangular channels, being kept within the minichannel regime, to perform at or above the level of a shell-and-tube heat exchanger in several metrics deemed important for the MSR PHX application. While engineers at ORNL were constrained to a shell-and-tube design when the MSBR was planned in the 1960's, additional considerations were required to assure that a compact design is feasible in the PHX application.

The work done at ORNL several decades ago was a great starting point for the design of a new system component, and several of the design considerations made in that study were also carried out in the PHX study laid out in this research. ANSYS Workbench and FLUENT allowed for rapid three-dimensional modeling and optimization of several design variables to arrive at the optimal solution for a slotted heat exchanger.

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Appendix A: ORNL Plant Diagrams

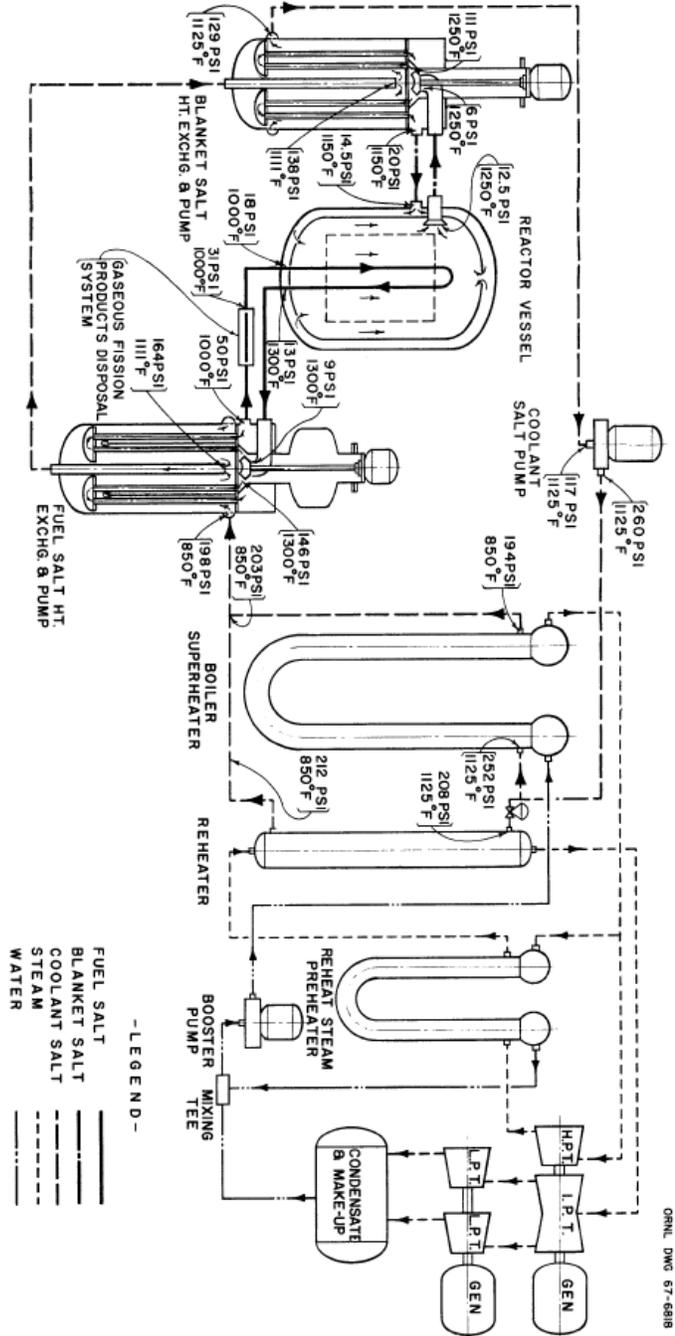


Figure 44: MSBR Plant diagram for Case B (Bettis, et al., 1967)

Appendix B: Parametric Mesh Study

Preliminary Mesh Study

Due to the simplistic nature of the geometry used in this study and limited computational resources available for preliminary studies, a parametric study was performed to test the minimum coarseness of mesh required to obtain accurate results from FLUENT.

Original mesh distributions were found to be unnecessarily close to the 2 million-node limit, and results within less than 4% were found with a 995% reduction in node count. The results from the parametric mesh study will be presented here.

On a 200mm length section of HX, with $N_W = 4$ and $N_H = 5$ computed by means of a model made with $N_W = 2$ and $N_H = 2.5$. A fine mesh was first developed in order to push the 2 million-node limit imposed on the 2GB RAM processor of the computer in use.

Table 22: Comparison of results for varying mesh counts on small-scale geometry.

| | Fine | Coarse |
|--------------------|-----------|-------------|
| Length (mm) | 200 | 200 |
| Number of nodes | 1,665,464 | 152,076 |
| Solid element size | 1.50E-04 | 1.20E-03 |
| Fluid element size | 3.50E-04 | 6.00E-04 |
| Sweep Divisions | 150 | 75 |
| Tin % difference | N/A | 0.164 |
| Tout % difference | N/A | 1.538 |
| h % difference | N/A | 3.946 |
| Pin % difference | N/A | 0.0521 |
| % node reduction | N/A | 995.1524238 |

A reduction of almost 1000% of nodes represents a tremendous decrease in computation times. At a much coarser distribution of nodes, results only 4% different are acceptable for preliminary studies. For this reason, the element size found for the coarse study will be kept throughout preliminary computations.

The results of this study are mesh-independent, with the results of this parametric study verifying the lack of a return for additional nodes above the given concentration.

Full-scale Mesh Study

The accuracy of full-scale final results is heavily dependent on the mesh that can be used on a larger application. With more than twice the number of channels compared to the preliminary study, the full-scale simulation requires significantly more nodes in its mesh. Channel lengths were increased to 750 mm in some full-scale simulations, which requires significantly more nodes than the 200 mm length used in preliminary studies. As mesh element size decreases, accuracy is gained in results. Fine meshes can be prohibitively resource-intensive with limiting computing power, so mesh sizes and distributions must be chosen judiciously. This section will outline the mesh used in full-scale studies (which was taken from the results from the preliminary study above), a coarser mesh that provided incorrect results, and a very fine mesh that would be preferred if computational power were increased.

The mesh shown in Figure 45 was used in the final optimized slotted HX design in this study.

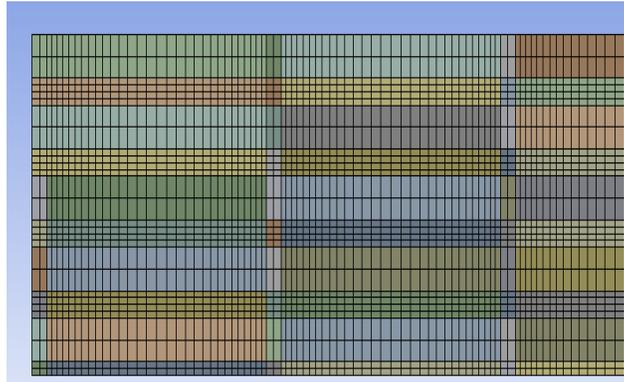


Figure 45: Mesh used in full-scale study that reached approximately 1,300,000 nodes.

Figure 46 shows a mesh with significantly less nodes that was used to validate the mesh size and distribution used in the full-scale study.

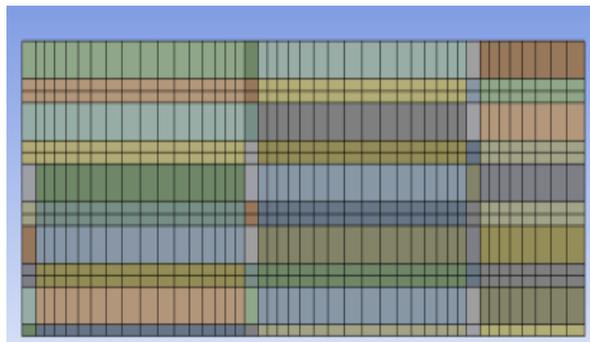


Figure 46: Course mesh that contained only approximately 250,000 nodes.

Since the results for the full-scale channel array was taken from a previous mesh study performed on a smaller scale. This process was performed to assure that scaling would not interfere with the accuracy of results. Table 23 shows the difference between results rendered for simulations of similar boundary conditions and geometry with the two different mesh distributions.

Table 23: Full-scale mesh comparison results.

| | W (# divisions) | H (# divisions) | L (# divisions) | Solid Divisions | # Nodes | % Error in Po |
|------------------------|---------------------------|---------------------------|---------------------------|----------------------------------|----------------|--------------------------------|
| Fine | 30 | 4 | 500 | 2x2 | 1,191,378 | ~5% |
| Coarse | 16 | 2 | 250 | 1x1 | 165,660 | ~35% |
| Desired (Very fine) | 60 | 10 | 1500 | 4x4 | 4,000,000+ | ~1% |

Just as a significant change was found between the two mesh distributions shown in Table 23, coarser meshes are expected to provide even more accurate results. The 2.5x4.5 array of channels was deemed sufficiently large to approximate results from a full-scale array of channels, but available computer imposed a strict restriction in mesh node count. The tradeoff between capturing more channels to more appropriately simulate a large array of channels caused mesh element sizes to be raised. Future studies using more powerful computers could use mesh sizes like those seen in Figure 47.

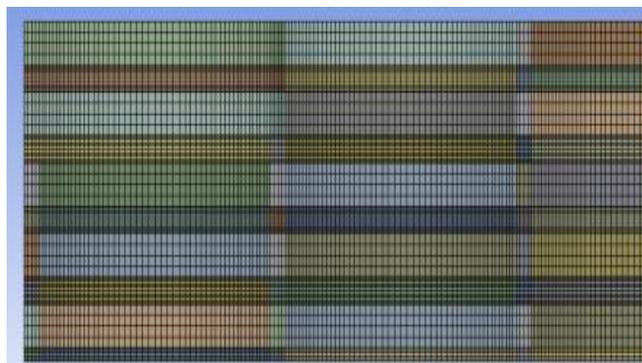


Figure 47: Desired mesh distribution that contains an estimate of approximately 4,000,000 nodes.

Appendix C: Preliminary Study Geometry Calculations

Surface Area Calculation

$$SA_{\text{slot}} = N_W N_H L [2W + 2H]$$

$$SA_{\text{circles}} = N_W N_H \pi D L$$

Volume Calculation

$$\text{Vol}_{\text{slot}} = [N_W W + SW(N_W + 1)][N_H H + SH(N_H + 1)][L]$$

$$\text{Vol}_{\text{circles}} = [N_W D + SW(N_W + 1)][N_H D + SH(N_H + 1)][L]$$

Material Calculation

$$\text{Mat}_{\text{slot}} = \text{Vol} - [HWN_W N_H L]$$

$$\text{Mat}_{\text{circles}} = \text{Vol} - \left[\frac{\pi D^2}{4} N_W N_H L \right]$$

Appendix D: ORNL Shell-and-tube Calculations

The values pertaining to the shell-and-tube PHX designed by ORNL can be found in several places. Whereas dimensions such as height, values such as the number of tubes, and parameters such as heat transfer area can be found in HX design documentation (Bettis, et al., 1967), the volume and material of each type of component inside the heat exchanger (floating heads, tubesheets, etc) must be estimated from other paperwork.

Sources were found for a single-fluid MSR's PHX, which had precise material requirements for a very similar heat exchanger (Robertson R. C., 1971).

Surface Area Calculation

$$SA_{\text{total shell-and-tube}} = 12,230\text{ft}^2 = 1,136\text{m}^2$$

Volume Calculation

$$VOL_{\text{total shell-and-tube}} = 22\text{ft} * \frac{\pi}{4} (6.5\text{ft})^2 = 730\text{ft}^3 = 20.7\text{m}^3$$

Material Calculation

$$MAT_{\text{total shell-and-tube}} = MAT_{\text{tubes}} + MAT_{\text{shell}} + MAT_{\text{tubesheets}} + MAT_{\text{floating head}} =$$

$$0.9547\text{m}^3 + 0.9285\text{m}^3 + 7.63 + 0.0278\text{m}^3 = 9.54\text{m}^3$$

$$MAT_{\text{tubes}} = MAT_{\text{inner tubes}} + MAT_{\text{outer tubes}} =$$

$$4347 * 15.3\text{ft} * \frac{\pi}{4} \left[\left(\left(0.375\text{in} * \frac{1\text{ft}}{12\text{in}} \right)^2 \right) - \left(\left(0.305\text{in} * \frac{1\text{ft}}{12\text{in}} \right)^2 \right) \right] +$$

$$3794 * 16.7\text{ft} * \frac{\pi}{4} \left[\left(\left(0.375\text{in} * \frac{1\text{ft}}{12\text{in}} \right)^2 \right) - \left(\left(0.305\text{in} * \frac{1\text{ft}}{12\text{in}} \right)^2 \right) \right] =$$

$$33.7163\text{ft}^3 = 0.9547\text{m}^3$$

$$MAT_{\text{shell}} =$$

$$22.2\text{ft} * \frac{\pi}{4} \left[\left(\left(68.7\text{in} * \frac{1\text{ft}}{12\text{in}} \right)^2 \right) - \left(\left(66.7\text{in} * \frac{1\text{ft}}{12\text{in}} \right)^2 \right) \right] =$$

$$32.7891\text{ft}^3 = 0.9285\text{m}^3$$

$$MAT_{\text{tubesheets}} \approx \frac{\text{mass}}{\text{density}} = \frac{149,100\text{lb}}{1} * \frac{0.4536\text{kg}}{1\text{lb}} * \frac{1\text{m}^3}{8,860\text{kg}} = 7.63\text{m}^3$$

$$MAT_{\text{floating head}} =$$

$$3.75\text{in} * \frac{1\text{ft}}{12\text{in}} * \frac{\pi}{4} (2\text{ft})^2 =$$

$$0.9817\text{ft}^3 = 0.0278\text{m}^3$$

Output Parameter Calculation

$$\left(\frac{SA}{VOL} \right)_{\text{shell-and-tube}} = \frac{1,136 \text{ m}^2}{20.7 \text{ m}^3} = 55 \frac{\text{m}^2}{\text{m}^3}$$

$$\left(\frac{SA}{MAT} \right)_{\text{shell-and-tube}} = \frac{1,136.2 \text{ m}^2}{9.54 \text{ m}^3} = 119 \frac{\text{m}^2}{\text{m}^3}$$

Appendix E: Material Properties

Hastelloy N

Composition (weight %) (Haynes International, 2002)

Table 24: Constituent elements in Hastelloy N.

| Nickel ¹ | Chromium | Molybdenum | Iron | Silicon | Manganese | Carbon | Co | Cu | W | Al+Ti |
|---------------------|----------|------------|------|---------|-----------|--------|-------|-------|-------|-------|
| 71 | 7 | 16 | 5* | 1* | 0.80* | 0.08* | 0.20* | 0.35* | 0.50* | 0.35* |

1 – Balance * - Maximum

Average Physical Properties

Table 25: Material properties of Hastelloy N (Haynes International, 2002).

| Physical Property | Temp, °C | Metric Units |
|---------------------------------------|------------|-------------------------------|
| Density | 22 | 8.86 g/cm ³ |
| Melting Range | 1300-1400 | -- |
| Mean Coefficient of Thermal Expansion | 21-204 | 11.6 x 10 ⁻⁴ m/m-K |
| | 21-316 | 12.3 x 10 ⁻⁴ m/m-K |
| | 21-427 | 12.7 x 10 ⁻⁴ m/m-K |
| | 21-538 | 13.4 x 10 ⁻⁴ m/m-K |
| | 21-649 | 14.0 x 10 ⁻⁴ m/m-K |
| | 21-760 | 14.7 x 10 ⁻⁴ m/m-K |
| | 21-871 | 15.3 x 10 ⁻⁴ m/m-K |
| Thermal Conductivity | 21-982 | 15.8 x 10 ⁻⁴ m/m-K |
| | 100 | 11.5 W/m-K |
| | 200 | 13.1 W/m-K |
| | 300 | 14.4 W/m-K |
| | 400 | 16.5 W/m-K |
| | 500 | 18.0 W/m-K |
| | 600 | 20.3 W/m-K |
| Specific Heat | 700 | 23.6 W/m-K |
| | 100 | 419 J/Kg-K |
| | 200 | 440 J/Kg-K |
| | 300 | 456 J/Kg-K |
| | 400 | 469 J/Kg-K |
| | 480 | 477 J/Kg-K |
| | 540 | 485 J/Kg-K |
| | 570 | 523 J/Kg-K |
| | 590 | 565 J/Kg-K |
| | 620 | 586 J/Kg-K |
| | 660 | 582 J/Kg-K |
| 680 | 578 J/Kg-K | |
| 700 | 578 J/Kg-K | |

Fuel Salt

Table 26: Fuel salt properties (Cantor, Cooke, Dworkin, Robbins, Thoma, & Watson, 1968).

| | Unit | Fuel Salt |
|-----------------------|--|--|
| Reference Temperature | °F | 1150 |
| Composition | mole % | $\frac{7}{3}\text{LiF}-\text{BeF}_2-\text{UF}_4$ |
| Molecular Weight | -- | 34 |
| Liquidus Temperature | °F | 842 |
| Density | lb/ft ³ | 127±6 |
| Viscosity | lb ft ⁻¹ hr ⁻¹ | 27±3 |
| Thermal Conductivity | Btu hr ⁻¹ lb ⁻¹ °F ⁻¹ | 1.5 |
| Heat Capacity | Btu lb ⁻¹ °F ⁻¹ | 0.55±0.14 |

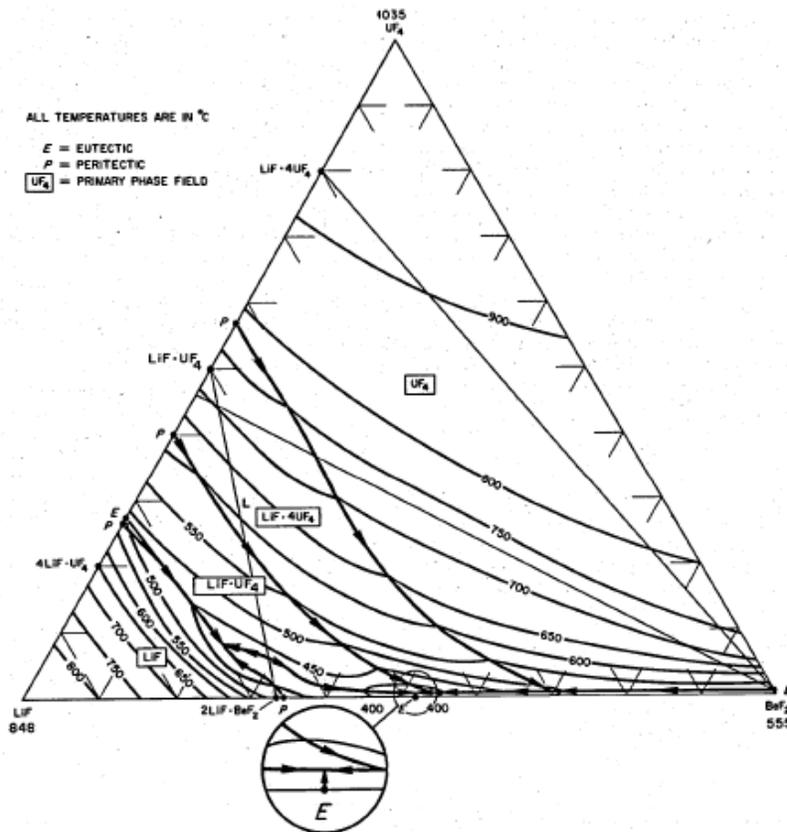


Figure 48: Fuel salts eutectic compositions of LiF, BeF₂, and UF₄, courtesy of Oak Ridge National Laboratory (Cantor, Cooke, Dworkin, Robbins, Thoma, & Watson, 1968).

Coolant Salt

(Sohal, Sabharwall, Sharpe, & Ebner, 2010)

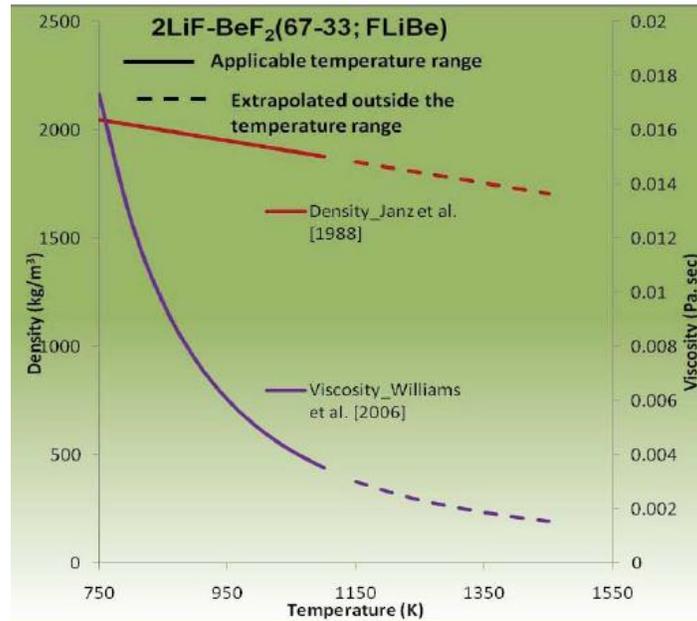


Figure 49: FLiBe properties, courtesy of Idaho National Laboratory (Sohal, Sabharwall, Sharpe, & Ebner, 2010).

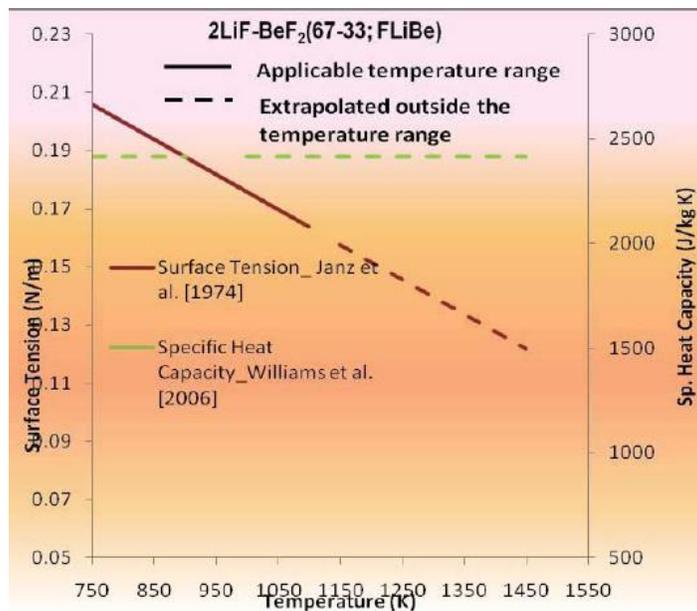


Figure 50: FLiBe properties, courtesy of Idaho National Laboratory (Sohal, Sabharwall, Sharpe, & Ebner, 2010).

Appendix F: Laminar-Turbulent Transition Discussion

The laminar-turbulent transition remains a relatively unknown science that has garnered significant research in an effort to find the physics occurring during the process.

For two parallel plates ($\alpha_c=0$), the laminar-turbulent has been found to occur at a Reynolds value of 2500. Square channels ($\alpha_c=1$), on the other hand, have been shown to transition at a Reynolds number of only 200. For any value between these two end data points, linear interpolation can be used to find the appropriate transition value (Kandlikar, Garimella, Li, Colin, & King, 2006). Equation 47 shows how the transition Reynolds number can be calculated according to the channel aspect ratio, α_c .

$$\mathbf{Re_t = 2500 - 300\alpha_c} \qquad \text{Equation 47}$$

Appendix G: Velocity and Temperature Contours

Temperature profile at half channel width along length of channel

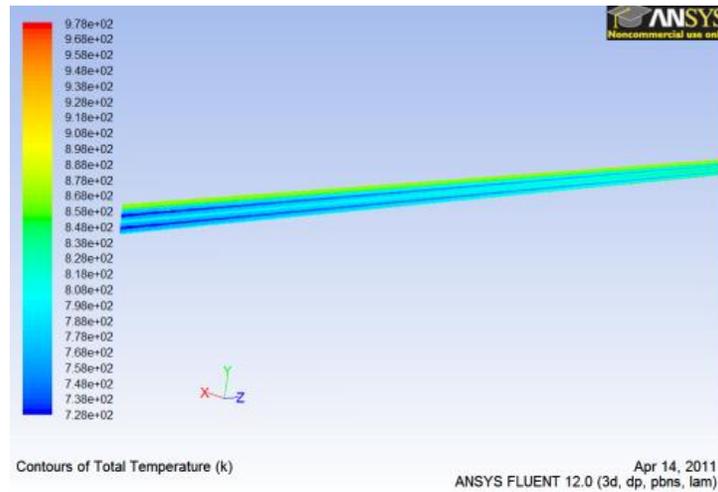


Figure 51: Temperature profile from cold inlet to a channel length of 0.25 m.

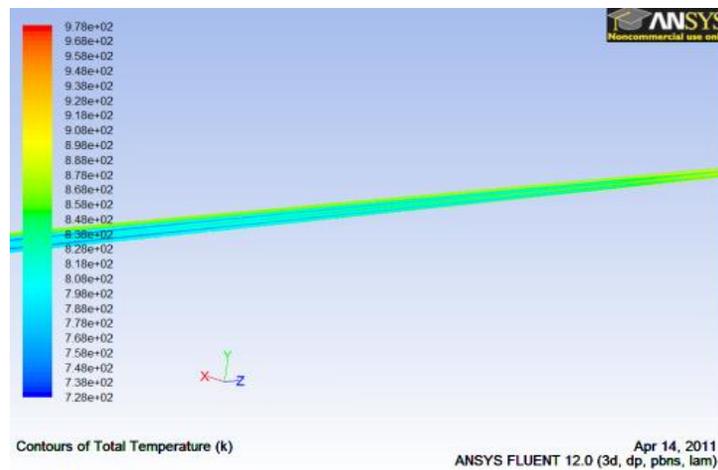


Figure 52: Temperature profile from a channel length of 0.25 m to a channel length of 0.5 m.

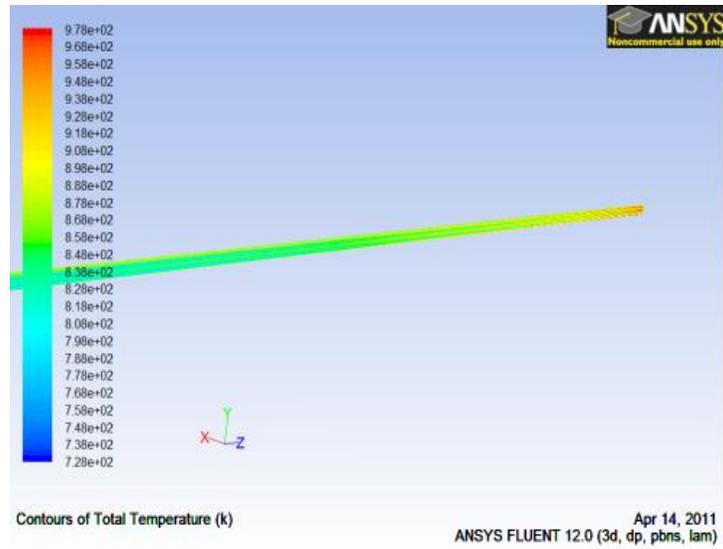


Figure 53: Temperature profile from a channel length of 0.5 m to a channel length of 0.75 m.

Temperature profiles at varying distances along channel length

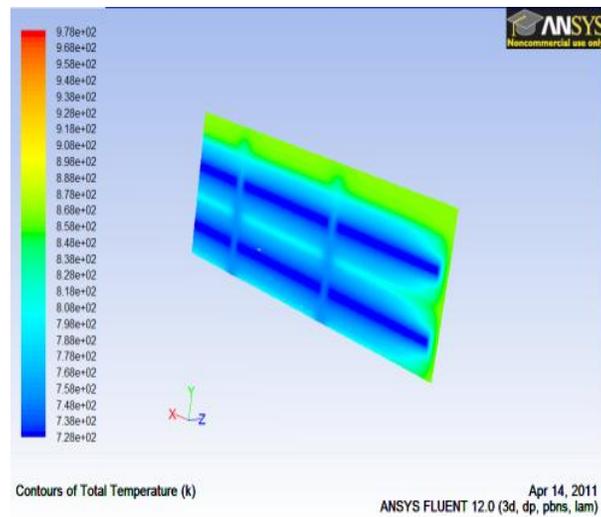


Figure 54: Temperature profile of geometry cross-section at cold inlet / hot outlet.

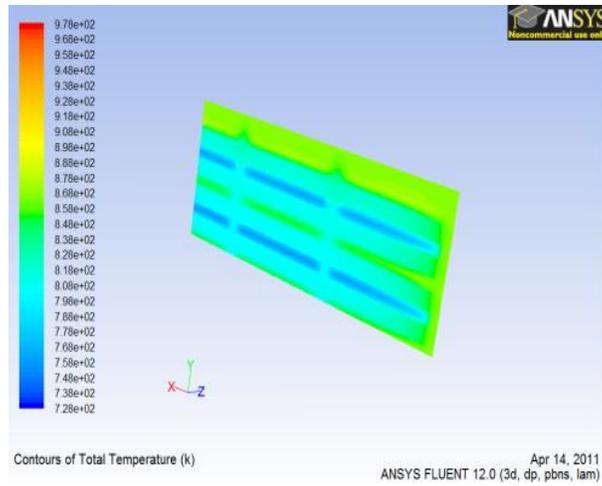


Figure 55: Temperature profile of geometry cross-section 0.25 m channel length from cold inlet / hot outlet.

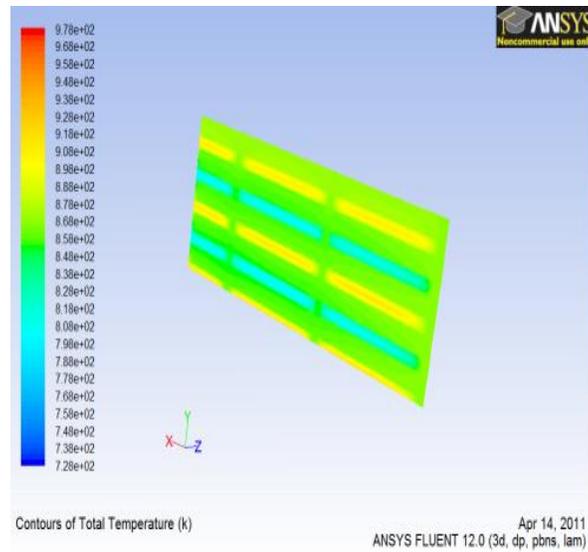


Figure 56: Temperature profile of geometry cross-section 0.5 m channel length from cold inlet / hot outlet.

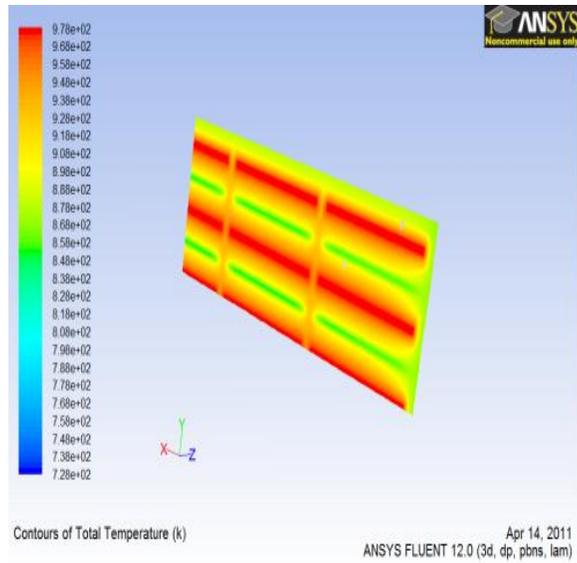


Figure 57: Temperature profile of geometry cross-section 0.75 m channel length from cold inlet / hot outlet.

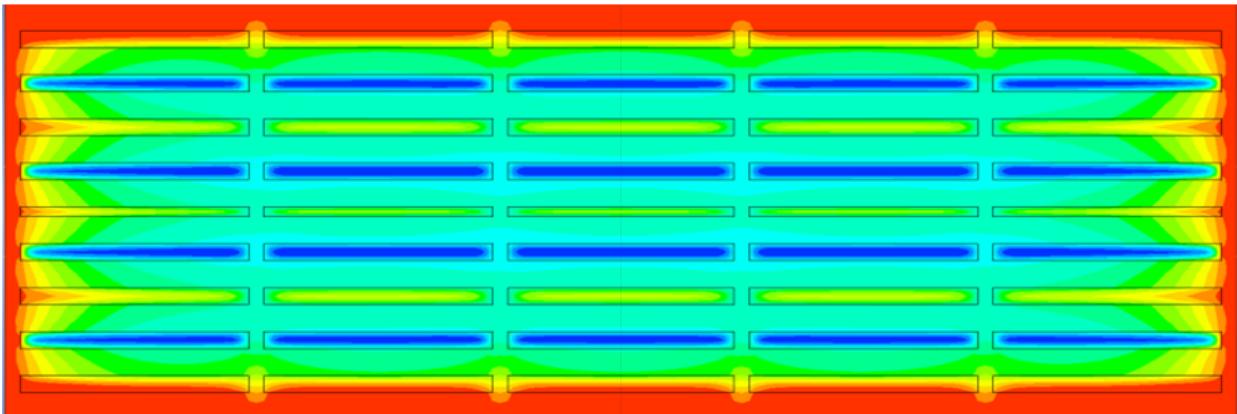


Figure 58: Temperature profile at half channel length, with Symmetry planes mirrored

Appendix H: Slotted HX Module Options

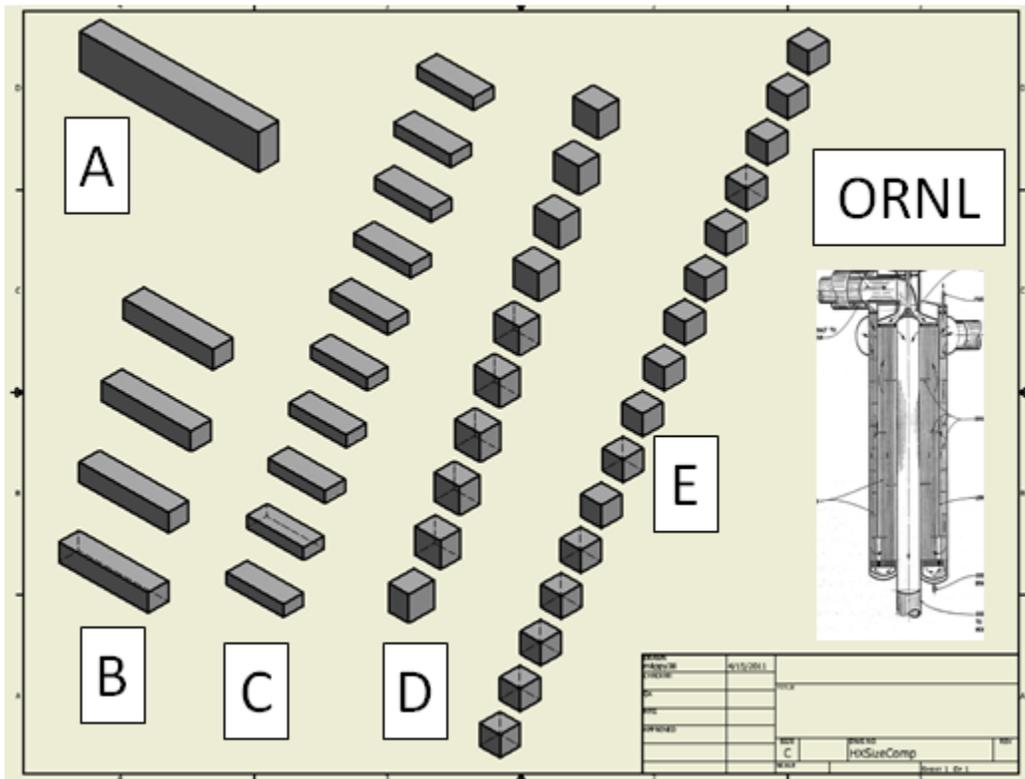


Figure 59: To-scale drawing of ORNL's PHX and several Slotted PHX options.

Table 27: Heat exchanger bank amount and size options portrayed in Figure 59.

| N (# banks) | N_W | N_H | H_{TOTAL} (m) | W_{TOTAL} (m) | L (m) | Figure 59 Key |
|------------------------------|-------------------------|-------------------------|---|---|------------------------|--------------------------------|
| 1 | 1,027 | 1,027 | 1.3 | 6.6 | 0.75 | A |
| 4 | 514 | 514 | 0.67 | 3.3 | 0.75 | B |
| 10 | 325 | 325 | 0.42 | 2.1 | 0.75 | C |
| 10 | 147 | 721 | 0.94 | 0.94 | 0.75 | D |
| 16 | 117 | 576 | 0.75 | 0.75 | 0.75 | E |